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AUTHOR Macek, Victor C.
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ABSTRACT

The nine Reactor Statics Modules are designed to introduce students to the use of numerical methods and digital computers for calculation of neutron flux distributions in space and energy which are needed to calculate criticality, power distribution, and fuel burnup for both slow neutron and fast neutron fission reactors. The last module, RS-9, includes a separate program, WANDIC (one-dimensional diffusion code with three energy-group representation), for the calculations of neutron spectra and group constants for use in fast breeder reactor calculations. It can be used with the criticality programs in RS-2 and RS-8. The reactor is divided into several concentric annular regions and it is assumed that those regions can be represented fairly well by their average properties. The previous modules show how these properties are found. The set of nine modules is intended to supplement textbooks and other lecture material generally available to students in their course work. It is assumed that students are familiar with elementary nuclear structure, neutron-nuclei interactions, and introductory material on fission chain reactors. (Author/SK)

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REACTOR STATICS MODULE, RS-9
MULTIGROUP DIFFUSION PROGRAM
USING AN EXPONENTIAL ACCELERATION TECHNIQUE

by

Victor C. Macek

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Reactor Statics Module, RS-9

Multigroup Diffusion Program Using an Exponential Acceleration Technique

9.1 Introduction

A reactor physicist or a nuclear engineer designing a power reactor needs to know the number of neutrons present within the system. This information is necessary for the calculation of the power distribution in a reactor from which further vital information on other related physical quantities needed to assure a successful operation of the reactor can be derived.

Previous modules discussed a broad area of problems which are encountered in reactor physics design. The basic input for calculations consists of engineering specifications and nuclear data. The former provide quantities like power density, fuel composition and a geometric description of the reactor. This information varies from one design to another. On the other hand, the nuclear data contains detailed cross-section information.

This input starts a rather long string of calculations before one can obtain results in a form which is suitable for engineering application.

The detailed treatment of space-energy physical phenomena in the entire reactor represents a very difficult task. Therefore, the problem is usually divided into several different stages and the flow of calculations passes from the detailed analysis of the neutron balance in an elementary fuel pin cell to the reactor as a whole consisting of assemblies which are represented by their average properties. In this procedure, the spatial and energy effects are reduced in detail; the former through the "homogenization", the latter through the group condensation. Of course, care must be taken in these simplifications. For example, neutron balance must be conserved.

The code WANDIC (one-dimensional diffusion code with a three energy-group representation) which is described in this module can be used to perform the global reactor calculations. The reactor is divided into several concentric annular regions and it is assumed that we can represent those regions fairly well by their average properties. The previous modules show how these properties are found.

Features of WANDIC

9.2 General Description

The program solves the three group diffusion equations

$$D_g \nabla^2 \phi^g - \Sigma_T \phi^g + S^g = 0 \quad \text{for } g = 1, 2, 3 \quad (9.2.1)$$

where the source term consisting of fission neutrons appears in the highest energy group only. The source terms in the lower groups are due to scattered neutrons from the adjacent upper group. The source S^g is given as

$$S^g = \frac{\delta_{1g}}{K} \sum_{g=1}^3 (v \Sigma_f)^g \phi^g + \Sigma_s^{g-1, g} \phi^{g-1} \quad (9.2.2)$$

where δ_{1g} is Kronecker's delta; $\delta_{1g} = 1$ for $g = 1$, zero otherwise. The group diffusion equations are solved for cylindrical geometry with θ and β symmetries by using a finite difference technique which is described in module RS-2.

Fluxes in each group and each mesh-point are calculated by solving a system of linear equations.

$$\begin{array}{r} b_1^g \phi_1^g - a_1^g \phi_2^g = S_1^g \\ \hline -c_n^g \phi_{n-1}^g + b_n^g \phi_n^g - a_n^g \phi_{n+1}^g = S_n^g \\ \hline -c_M^g \phi_{M-1}^g + b_M^g \phi_M^g = S_{M-1}^g \end{array} \quad (9.2.3)$$

where indices 1, M denote mesh-points at the center and outer boundary of the reactor respectively. S_n is the source at the point n.

In matrix form the system (9.2.3) can be written as

$$A_{\phi}^g = S^g \tag{9.2.4}$$

where ϕ^g is a vector representing fluxes ϕ_n^g at all mesh-points. The source term is a vector whose components are

$$S_n^g = h_n (\Sigma_{sn}^{g-1, g} \phi_n^{g-1} + \frac{\delta_{1g}}{K} \sum_{j=1}^3 (v \Sigma_f)_n^j \phi_n^j)$$

where h_n is volume of the mesh-interval. In matrix notation S_n^g can be written as

$$S^g = E^g \phi + \frac{\delta_{1g}}{K} F \phi, \tag{9.2.5}$$

where

δ_{1g} is the Kronecker delta and

$$\phi = \begin{bmatrix} \phi^1 \\ \phi^2 \\ \phi^3 \end{bmatrix} \quad \begin{aligned} E^g &= (E^{1g} \ E^{2g} \ 0) \\ F &= (F^1 \ F^2 \ F^3) . \end{aligned}$$

The vector ϕ^j is formed from the elements ϕ_n^j , the diagonal matrix E^{jg} is formed from the elements $\Sigma_{sn}^{g-1, g}$, and the diagonal matrix F^j is formed from the elements $(v \Sigma_{fn})^j$.

Finally, the assembly of group equations may also be regarded as a matrix problem, which may be written as

$$\underline{A} \underline{\phi} = \underline{S}, \tag{9.2.6}$$

where

$$\underline{\underline{A}} = \begin{bmatrix} A^1 & 0 & 0 \\ 0 & A^2 & 0 \\ 0 & 0 & A^3 \end{bmatrix} \quad \underline{\underline{S}} = \underline{\underline{E}} \phi + \frac{\delta l g}{k} \underline{\underline{F}} \phi$$

$$\underline{\underline{F}} = \begin{bmatrix} F^1 \\ F^2 \\ F^3 \end{bmatrix}^T \quad \underline{\underline{E}} = \begin{bmatrix} E^1 \\ E^2 \\ E^3 \end{bmatrix}^T$$

9.3 The Inner Iteration

The inner iteration allows calculation of spatial dependence of fluxes with respect to the boundary conditions. For simple one-dimensional geometries the inner iteration can be avoided and fluxes at each mesh-point calculated by a direct procedure such as Thomas algorithm which is described in Module RS-2, Chapter 2. This algorithm is used in this code.

9.4 The Outer Iteration

The overall matrix problem to be solved is expressed in Eq. (9.2.6) as

$$\underline{\underline{A}} \phi = \underline{\underline{S}} \tag{9.4.1}$$

in which

$$\underline{\underline{S}} = \underline{\underline{E}} \phi + \frac{1}{k} \underline{\underline{F}} \phi \tag{9.4.2}$$

The normal procedure is to begin the solution of the group equations in the highest energy group by using a flux guess for the calculation of the fission source. In the highest group there are no slowing-down scattering terms. Proceeding to the next group, the slowing-down part of the source can be calculated from the new fluxes in group 1; since fission source in this model appears only in the highest group, the fission part of the source in group 2 is zero. The calculation can proceed in this fashion to thermal energy. At the end of the outer iteration we have a completely new flux vector, so that the fission source $\underline{F} \phi$ may be recalculated and the whole process may be repeated.

The equation which must be solved is

$$(A - E)\phi = \frac{1}{k} F \phi. \quad (9.4.3)$$

The iteration process proceeds by defining a vector

$$\psi = \frac{1}{k} F\phi = \frac{1}{k} S \quad (9.4.4)$$

where

$$S = F\phi. \quad (9.4.5)$$

(Note that S denotes the fission source only.)

The iteration is defined by the relation

$$(A - E)\phi^{(p)} = \psi^{(p-1)}. \quad (9.4.6)$$

From Eq. (9.4.5) we can write

$$S^{(p)} = F\phi^{(p)}. \quad (9.4.7)$$

By combining Eqs. (9.4.5) and (1.4.7), the p-th iterate of the source reactor is given by

$$\frac{1}{k^{(p)}} S^{(p)} = \psi^{(p-1)}. \quad (9.4.7)$$

To estimate the eigenvalue one utilizes the fact that when a convergence is being approached, the fission source for one iteration should equal the fission source for the next. In this code the estimate of the eigenvalue is determined in the following fashion. The iteration starts with $k^{(0)}$ equal to 1 and $S^{(0)}$ such that

$$\int_{(\text{vol of core})} S^{(0)} dv = 1 \quad (9.4.8)$$

Then from (1.4.7) and (1.4.4),

$$k^{(1)} = \int_{(\text{vol of core})} S^{(1)} dv, \quad (9.4.9)$$

and similarly

$$k^{(p)} = \int S^{(p)} dv. \quad (9.4.9)$$

The iteration continues until convergence of the eigenvalue $k^{(p)}$ is obtained, i.e.,

$$\left| \frac{k^{(p)} - k^{(p-1)}}{k^{(p)}} \right| < \epsilon. \quad (9.4.10)$$

The single power iteration described above tends to converge rather slowly and acceleration techniques are used to improve the convergence rate. If we denote the unaccelerated fission source for iteration p by $S^{*(p)}$, then we calculate an accelerated fission source from

$$S^{(p)} = S^{(p-1)} + \omega(S^{*(p)} - S^{(p-1)}). \quad (9.4.11)$$

This form of acceleration is referred to as a first order acceleration. The Chebyshev polynomial method is described Reactor Statics Module, RS-8. The optimum value of ω requires a knowledge of the maximum eigenvalue of the matrix $(A - E)F^{-1}$. Generally this eigenvalue is not known. One of the methods to solve this problem is described in the following section.

9.5 The Exponential Over-Relaxation Technique

The program uses a special technique for the calculation of the source introduced for the subsequent iteration step.

Fission sources are computed from the calculated fluxes at each mesh point as follows

$$S_n^* = \sum_j (\nu \Sigma_f^j) \phi_n^j \quad \begin{matrix} j = 1, 2, 3 \\ n = 1, 2, \dots \end{matrix} \quad (9.5.1)$$

For the first iteration the initial source-guess will be, e.g.,

$$S_n^{(0)} = \frac{1}{\pi R^2}, \quad (9.5.2)$$

where R is the radius of the core. The source for the p-th iteration will be given by formula

$$S_n^{(p)} = S_n^{(p-1)} + \omega(S_n^* - S_n^{(p-1)}), \quad (9.5.3)$$

where ω is the relaxation factor whose optimal value lies between 1 and 2.

The equation (9.5.3) can be rewritten as

$$S_n^{(p)} = S_n^{(p-1)} \left[1 + \omega \left(\frac{S_n^*}{S_n^{(p-1)}} - 1 \right) \right] \quad (9.5.4)$$

$$S_n^p = S_n^{(p-1)} \exp \left[\omega \left(\frac{S_n^*}{S_n^{(p-1)}} - 1 \right) \right]$$

for $\left(\omega \frac{S_n^*}{S_n^{(p-1)}} - 1 \right) \ll 1$.

The optimal value of the relaxation factor ω is automatically computed during the calculation.

Letting,

$$Q' = \frac{S_n^*}{S_n^{(p-1)}} - 1, \quad (9.5.5)$$

we can have two cases:

1) $Q' > 0$.

The exponential function can be expressed as

$$\begin{aligned} \exp\left[\omega\left(\frac{S_n^*}{S_n^{(p-1)}} - 1\right)\right] &\approx 1 + \omega\left(\frac{S_n^*}{S_n^{(p-1)}} - 1\right) \\ &\approx 2 - \left(1 - \omega\left(\frac{S_n^*}{S_n^{(p-1)}} - 1\right)\right) \end{aligned} \quad (9.5.6)$$

$$\approx 2 - e^{-Q},$$

where

$$Q = \omega^* \left| \frac{S_n^*}{S_n^{(p-1)}} - 1 \right|. \quad (9.5.7)$$

2) $Q' < 0$.

In this case we have

$$\exp\left[\omega\left(\frac{S_n^*}{S_n^{(p-1)}} - 1\right)\right] = e^{-Q}. \quad (9.5.8)$$

The calculation of the exponential function in the core is speeded up by the following approximations:

$$\text{For } Q > 0.1 \quad e^{-Q} \rightarrow \exp[-Q]$$

$$0.001 < Q < 0.1 \quad e^{-Q} \rightarrow \frac{1 - Q/2}{1 + Q/2}$$

$$Q < 0.001 \quad e^{-Q} \rightarrow 1 - Q .$$

In order to find the optimum value of ω it is necessary to scan the maximum relative error at each iteration given by the expression

$$\epsilon^{(p)}(S) = \max_i \left(\frac{S_i^{(p)} - S_i^{(p-1)}}{S_i^{(p-1)}} \right) , \quad (9.5.9)$$

where i runs through all mesh-points. The computation is normally started with ω equal to 1.9. At the end of 20 iterations the convergence ratio is tested. If the convergence is fast enough, let us say, if

$$\frac{\epsilon^{(10)}(S)}{\epsilon^{(20)}(S)} > 1.5 , \quad (9.5.10)$$

the iteration with unchanged ω is continued for another 20 iterations. If the convergence is slow with respect to the above criterion, i.e., the inequality is reversed and if the relative point-wise error oscillates which can be described by the inequality

$$\left| \frac{\epsilon^{(20)}}{\epsilon^{(19)}} [1 + |\epsilon^{(19)}(S)|] \right| > 1.0, \quad (9.5.11)$$

ω is reduced by 0.1 and with this new value of ω the iteration process is continued in the same manner as described above.

9.6 Accuracy Criteria

In this code two criteria for the outer iteration are adopted. The convergence of the multiplication factor k will be reached when the following holds.

$$\left| \frac{k^{(p)} - k^{(p-1)}}{k^{(p)}} \right| < \frac{\epsilon_1}{5.0} , \quad (9.6.1)$$

where $k^{(p-1)}$ is the eigenvalue obtained in the previous iteration. Once criticality is obtained, the program proceeds to obtain the pointwise convergence of neutron group fluxes which will be reached when the following condition is fulfilled.

$$\left| \frac{\phi_n^{(p)} - \phi_n^{(p-1)}}{\phi_n^{(p-1)}} \right|_{\max} < \epsilon_2 . \quad (9.6.2)$$

With the poison search another convergence criterion must be included which is discussed further.

9.7 Control Search

The program does a control search on a poison or any fictitious material which adjusts the reactor to the critical condition. Let the multiplier of the poison concentration be denoted by X . In general, the parameter X can be applied to any other type of control. By changing X the program finds k , then changes X , finds k again, and so on. In the early stages of the process the k calculations stop when both criteria (9.6.1) and (9.6.2) are satisfied. Then the criterion to guarantee criticality is applied, i.e.,

$$|k(\text{converged}) - k_c| < \epsilon_1, \quad (9.7.1)$$

if k_c is chosen to be unity, the reactor is critical. Clearly k_c also can be chosen to take into account the presence of other reactivity controls.

The initial guess for the multiplication factor proceeds according to the following formulas:

- a) If the first guess for the multiplication factor X is zero, the second guess is

$$X_2 = \Delta X \{ \text{of sign of } (1.0 - k_1) \} . \quad (9.7.2)$$

- b) If the first guess is non-zero the second guess is given by

$$X_2 = 1.0 + \Delta X \{ [\text{sign of } (1.0 - k_1)] \} * X, \quad (9.7.2)$$

where k is the converged eigenvalue and ΔX is an increment in the

parameter X. This formula tries to ensure that X moves in the right direction: it assumes that if X is negative, an increase of X decreases the reactivity, and the converse of this if X is positive (this would be applied, e.g., for the search of fissile isotopic composition). The program finishes when three conditions (9.6.1), (9.6.2), and (9.7.1) are satisfied.

With the quantities X_1 and X_2 the search is begun and X is varied in order to make k equal to unity. It is easy enough then to make a simple, search procedure such as ordinary "regula falsi," but these processes appear to be very inefficient in this context. The difficulty is that given X, we can find k but the more accurately we require it, the greater the number of iterations is required. One way to go is to improve "regula falsi" process so that the search is accelerated. In ordinary "regula falsi" at each stop, it is one of the limits (a, b) which is used and the last approximation of the root. Instead of that we can use the last two approximations which are closer to the root which is searched for than the limits of the interval (a, b). The formula which is used has the form given by Equation (9.7.5) and is illustrated in Figure 1.

$$a_{n+1} = a_n - f(a_n) * \frac{a_n - a_{n-1}}{f(a_n) - f(a_{n-1})} . \quad (9.7.4)$$

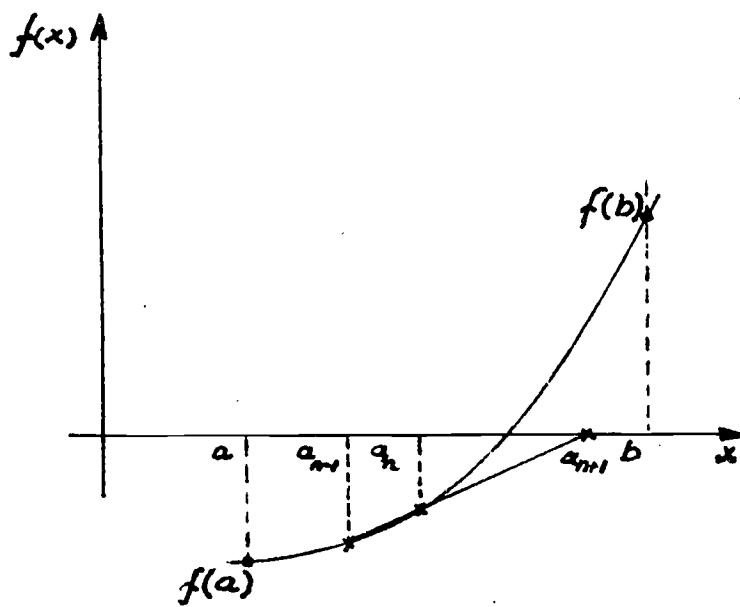


Fig. 1.

The a_1 is found from the formula

$$a_1 = b - f(b) \frac{b - a}{f(b) - f(a)}. \quad (9.7.5)$$

Depending on the sign of $f(a)$, $f(b)$, and $f(a_1)$, a_2 is computed from the formula (9.7.6) if $f(a) < 0$, $f(b) > 0$ and $f(a_1) < 0$; if $f(a_1) > 0$ from the formula (9.7.7),

$$a_2 = a_1 - f(a_1) \frac{b - a_1}{f(b) - f(a_1)} \quad (9.7.6)$$

and

$$a_2 = a_1 - f(a_1) \frac{a_1 - a}{f(a_1) - f(a)}. \quad (9.7.7)$$

If perchance the point a_3 will appear outside the interval (a, b) then in the following step it will be necessary to consider the closer limit instead of the point a_3 . It can be shown that convergence of this method is faster than the ordinary "regular falsi".

9.8 Power Normalization of Neutron Fluxes

The source-normalized power at the point \vec{r} is given by the expression

$$p(\vec{r}) = \sum_{j=1}^3 \kappa_j \Sigma_f^j(\vec{r}) \phi^j(\vec{r}) \quad j = 1, 2, 3 \quad (9.8.1)$$

where κ_j stands for energy produced per fission reaction, $\phi^j(\vec{r})$ is the source-normalized flux. In order to get fluxes normalized to the total power output of the reactor we have to multiply the source normalized fluxes by the normalization factor γ given by the formula

$$\gamma = \frac{P|H}{R \int_0^R p(\vec{r}) d\vec{r}}, \quad (9.8.2)$$

where $d\vec{r} = 2\pi r dr$ and H is the height of the reactor. The power-normalized flux ϕ_j' is then given by

$$\phi_j'(\vec{r}) = \gamma \phi_j(\vec{r}). \quad (9.8.3)$$

9.9 Organization of the Program

The code WANDIC has the following limitations:

- a) Number of energy groups is three
- b) The reactor can be divided into four regions from which the outer one is non-multiplicative. If use of fewer regions is desired, the the entry data for some of these regions will be identical.
- c) Memory array space is provided for up to 300 mesh-points.
- d) Number of intervals dividing each region must be even.

The code consists of a number of subprograms which perform different parts of calculation and are controlled by the main program.

9.10 The Role of Different Subprograms

PLOT: The subprogram participating in preparation of the output and plotting the related results on the printer.

SIMP: The subprogram performing the integration of functions over the core volume by SIMPSON'S rule.

ZERO: The subprogram which employs the accelerated "regula falsi" technique.

9.11 Flow-Chart

Besides its role as the manager of subprograms, the main program is in charge of entry of data for calculation of fluxes and preparation of results for the output. In Figure 2 the flow-chart of the sequence of calculation is presented.

FLOW-CHART A

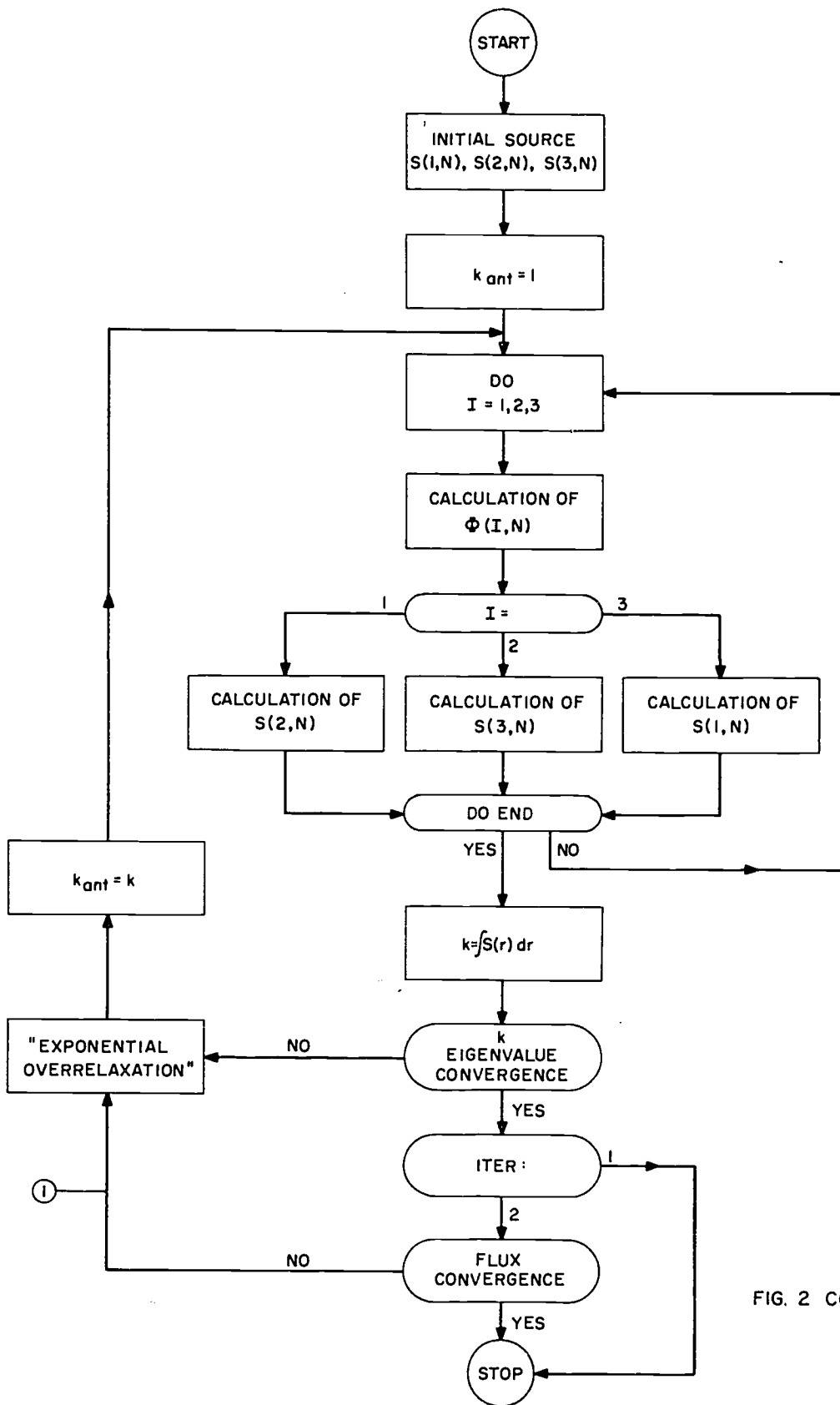


FIG. 2 CON'T.

9.12 Output

The output of the program are:

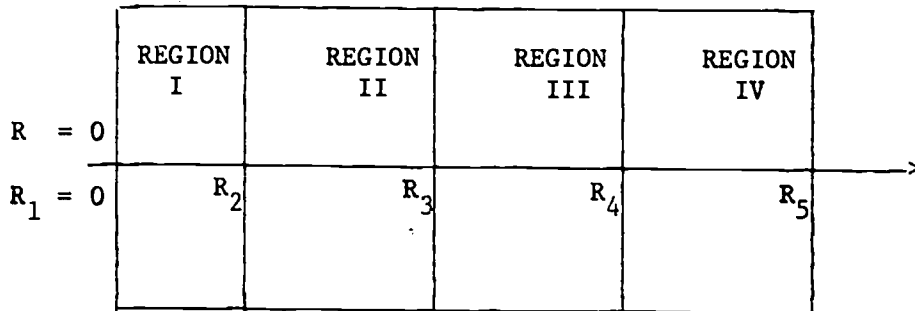
- a) source normalized group fluxes versus radius
- b) relative magnitudes of fluxes at each mesh-point with respect to the flux at the symmetry axis of the core
- c) ratio of group fluxes
- d) eigenvalue of the unpoisoned reactor
- e) poison concentration needed for criticality
- f) power normalized group fluxes
- g) power generated in each fueled region with respective power densities
- h) for user who wants to watch the iteration process, printing of eigenvalue K computed at each iteration is provided.

9.13 Names of Principal Variables Used by the Program

- H(K): mesh-size of the region K
- FLUX(I,N): neutron flux of the group I at the mesh-point N
- SDS(N): slowin_g-down source at the mesh-point N
- FS(N): fission source at the mesh-point N
- S(N): source at the mesh-point N
- EPS(P): maximum relative error for the source on the p-th iteration
- FME: $K_c = 1$
- XP: multiplication factor for the control poison cross-section
- FMANT: eigenvalue of the preceding iteration
- FPR(K): power density of the region K.

9.14 Use of the Program for a Simple Problem. Presentation of Entry of Data

The data introduced here are for illustration purposes only. Consider a cylindrical reactor consisting of three different fissionable regions and a reflector (see figure below). The outer radius of the first region is $R_2 = 87.7$ cm, the second $R_3 = 124.00$ cm, $R_4 = 152.00$ cm, $R_5 = 159$ cm.



The data for the problem would start:

1 card: symbols used for reading are

```
OVREL CONV PREC BH2 SCP POW NINT(1) NINT(2) NINT(3)
NINT(4) ICP ICC
```

where

- OVREL: the relaxation factor (Sec. 1.5). This factor for a new problem is put equal to 1.9. Once an "optimum" factor is found, this may be introduced into a set of similar problems (e.g., with the same mesh-point distribution).
- CONV: the pointwise flux convergence accuracy number (ϵ_2). (See Eq. (9.6.2))
- PREC: the accuracy (ϵ_1) for the eigenvalue convergence. (See Eq. (9.6.1)).
- BH2: axial buckling.
- SCP: microscopic cross-section of a poison material for the third group energy level in barns.
- POW: thermal output of the reactor in watts.

NINT(K): number of intervals per region K.

ICP: control search parameter. If its value is
 1...control poison is considered in region 1
 2...region 2
 3...region 3
 4...all fueled regions
 5...regions 1 and 2
 6...regions 2 and 3

ICC: is the case control parameter (equal to zero, if only one case is to be treated; positive, if otherwise)

The three group diffusion equations in one of the multiplicative powers assumes, in general, the following form

$$D_1 \Delta^2 \phi_1 - (\Sigma_{a_1} + \Sigma_{S_1^{1+2}}) \phi_1 + \sum_{i=1}^3 \frac{(\nu \Sigma_f)_i \phi_i}{k} = 0$$

$$D_2 \Delta^2 \phi_2 - (\Sigma_{a_2} + \Sigma_{S_1^{2+3}}) \phi_2 + \Sigma_{S_1^{1+2}} \phi_1 = 0$$

$$D_3 \Delta^2 \phi_3 - \Sigma_{a_3} \phi_3 + \Sigma_{S_1^{2+3}}^1 \phi_2 = 0$$

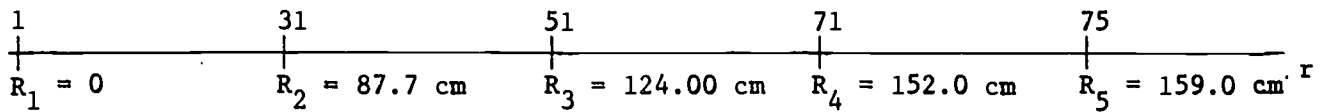
For example, with calculated macroscopic cross-sections

$$1.7 \Delta^2 \phi_1 - (0.004 + 0.06) \phi_1 + \frac{1}{k} (0.43 \phi_1 + 0.015 \phi_2 + .16 \phi_3) = 0$$

$$0.85 \Delta^2 \phi_2 - (0.025 + 0.05) \phi_2 + 0.06 \phi_1 = 0$$

$$0.39 \Delta^2 \phi_3 - 0.13 \phi_3 + 0.05 \phi_2 = 0$$

Let us divide the region I into 30 equal intervals, the regions II and III into 20 intervals and region IV into 4 intervals. The mesh-points in the radial direction will be numbered as shown in the figure below.



Let us assume that the axial buckling $BH2 = 1.56 * 10^{-3} \text{ cm}^2$; we shall use poison search in all fissionable regions, $ICP = 4$, the microscopic absorption cross-section of the poison material $\sigma = 2109 \text{ bn}$, and this will be the only case to be treated. The power output of the reactor is 2441 MW.

OVREL	CONV	PREC	BH2	SCP	POW
1.9E - 01	1.E - 02	1.E - 04	156.E - 05	2109.E 00	2441.E + 06
N_1	N_2	N_3	N_4	ICP	ICC
30	20	20	04	04	0

2, 3, 4th Cards:

The second card carries physical data for the first region--group 1, the third those for group 2, the fourth for group 3. Data are read in the following order (the symbols are self-explanatory; means $\frac{\text{watts}}{\text{fission}}$).

$$D_j \quad \Sigma_{aj} \quad \Sigma_{r_{1j \rightarrow j+1}} \quad (v\Sigma_f)_j \quad (\Sigma_f)_j$$

2nd:	17.E - 1	4.E - 3	6.E - 2	43.E - 2	93.E - 15
3rd:	85.E - 2	25.E - 3	5.E - 2	1.E - 2	19.E - 14
4th:	39.E - 2	13.E - 2	0.E + 0	16.E - 2	21.E - 13

Other 6 cards carry the similar data for regions II and III.

11, 12, 13th Cards:

These cards carry the data for the outer non-fissionable region in order of groups 1, 2, 3. These data are

$$D_j \quad \Sigma_{aj} \quad \Sigma_{S j \rightarrow j+1}$$

Let the reflector have the following data in order of groups 1, 2, 3

11th:	19.E - 1	36.E - 5	26.E - 3
12th:	92.E - 2	57.E - 5	28.E - 3
13th:	31.E - 2	85.E - 4	0.E 0

14th Card:

This card carries outer radii of regions with increasing order

R_2	R_3	R_4	R_5
877.E - 1	124.E 0	152.E 0	159.E 0

15th Card:

The value carried by this card is the increment of the multiplier adjusting the poison concentration at early stages of control search.

PP

0.05

9.15 Rules for Card Punching

The format of data read is fixed: real variables are read with the format E 10.0, the integer ones with the format I2 with one blank space separating them. On the last card PP, variable is read with the format F5.3.

```

C      MAIN PROGRAM                                A  1
C      PROGRAM WANDIC                              A  2
C                                                    A  3
C                                                    A  4
C      THE PROGRAM SOLVES THE THREE-GROUP DIFFUSION EQUATIONS A  5
C      FOR A CYLINDRICAL REACTOR CONSISTING OF THREE FUELED REGIONS A  6
C      AND A REFLECTOR                              A  7
C                                                    A  8
C                                                    A  9
C      INPUT-MEANING OF SYMBOLS                    A 10
C                                                    A 11
C      NINT...NUMBER OF INTERVALS FOR EACH REGION A 12
C      OVREL...OVERRELAXATION FACTOR              A 13
C      CONV...PRECISION FOR THE POINTWISE FLUX CONVERGENCE A 14
C      X...MULTIPLIER FOR THE CONTROL SEARCH PARAMETER A 15
C      D...DIFFUSION COEFFICIENT                  A 16
C      SCA...MACROSCOPIC ABSORPTION CROSS-SECTION A 17
C      XNF...MACROSCOPIC FISSION CROSS-SECTION*NU A 18
C      XKF...MACROSCOPIC FISSION CROSS-SECTION*KAPPA A 19
C      SR...SCATTERING CROSS-SECTION INTO THE LOWER ADJACENT GROUP A 20
C      BH2...TRANSVERSE BUCKLING                  A 21
C      RR...OUTER RADIUS OF THE REGIONS 1,2,3,4   A 22
C      POW...POWER OUTPUT OF THE REACTOR          A 23
C      SCP...MICROSCOPIC CROSS-SECTION OF THE CONTROL POISON A 24
C      ICP...CONTROL PARAMETER FOR THE POISON SEARCH A 25
C      1...REGION 1                                A 26
C      2...REGION 2                                A 27
C      3...REGION 3                                A 28
C      4...ALL FUELED REGIONS                      A 29
C      5...REGIONS 1&2                             A 30
C      6...REGIONS 2&3                             A 31
C      ICC...PROGRAM CONTROL INDICATING THE NUMBER OF CASES A 32
C      POSITIVE INTEGER...SEVERAL CASES TO BE TREATED A 33
C      0...ONLY ONE CASE                          A 34
C                                                    A 35
C                                                    A 36
C      LIMITATIONS OF THE CODE                    A 37
C                                                    A 38
C      1ST:NINT(K) MUST BE EVEN NUMBER             A 39
C      2ND: TOTAL NUMBER OF MESH-POINTS BE LESS THAN 301 A 40
C                                                    A 41
0001      SUBROUTINE PLOT (LI,N,LP)                 A 42
0002      REAL MX,MY,PRNT(6,9),DISP(6),DY,X(300),Y(3,300),H(4) A 43
0003      INTEGER LI,LP,PT,STRG(101),BLK,PER(21),AST,I,N,NINT(4) A 44
0004      COMMON Y,H,NINT                          A 45
0005      DATA PRNT/4HFAST,4H FLU,4HX -,4HGROU,4HP 1 ,1H ,4HREF30,4HNANC,4HE A 46
      1 FL,4HUX -,4HGROU,4HP 2 ,4HLOW,4H FLU,4HX -,4HGR,4HP 3 ,1H ,4H A 47
      2GROU,4HP 1 ,1H ,1H ,1H ,1H ,4HGROU,4HP 2 ,1H ,1H ,1H ,1H ,4HGROU,4 A 48
      3HP 3 ,1H ,1H ,1H ,1H ,4HGROU,4HP 1 ,4H/ GR,4HOUP ,4H3 ,1H ,4HGRO A 49
      4U,4HP 2 ,4H/ GR,4HOUP ,4H3 ,1H ,4HGROU,4HP 1 ,4H/ GR,4HOUP ,4H2 A 50
      5 ,1H /,8LK,AST/1H ,1H*/ ,PER/21*.'/' A 51
0006      DO 1 I=1,101                             A 52
0007      STRG(I)=BLK                                A 53
C                                                    A 54
C      CALCULATION OF RADII CORRESPONDING TO THE MESH-POINTS A 55
C                                                    A 56
0008      X(1)=0.0                                  A 57
0009      K=1                                         A 58

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0010          DO 2 I=2,N                      A 59
0011          X(I)=X(I-1)+H(K)                A 60
0012          IF (I.EQ.N/INT(K)) K=K+1        A 61
0013          2 CONTINUE                      A 62
0014          WRITE (6,9) (PRNT(I,LP),I=1,6)  A 63
0015          MN=Y(LI,1)                      A 64
0016          MX=MN                          A 65
0017          DO 6 I=2,N                      A 66
0018          IF (MX-Y(LI,I)) 3,4,4          A 67
0019          3 MX=Y(LI,I)                    A 68
0020          GO TO 6                          A 69
0021          4 IF (MN-Y(LI,I)) 6,6,5        A 70
0022          5 MN=Y(LI,I)                    A 71
0023          6 CCONTINUE                     A 72
0024          DY=(MX-MN)/100                 A 73
0025          DO 7 I=1,6                     A 74
0026          7 DISP(I)=MN+(I-1)*20*DY      A 75
0027          WRITE (6,10) DISP              A 76
0028          WRITE (6,11) (PER(I),I=1,21)  A 77
0029          DO 8 I=1,N                     A 78
0030          PT=(Y(LI,I)-MN)/DY+1.5        A 79
0031          STRG(PT)=AST                   A 80
0032          WRITE (6,12) X(I),STRG        A 81
0033          8 STRG(PT)=BLK                 A 82
0034          RETURN                          A 83
C                                           A 84
0035          9 FORMAT (1H1,45X,7HPLOT OF,1X,6A4,/,57X,6HZ-AXIS) A 85
0036          10 FORMAT (11X,6(E10.3,10X))  A 86
0037          11 FORMAT (16X,21(A1,4X),/,7H R-AXIS) A 87
0038          12 FORMAT (E11.3,5X,101A1)    A 88
0039          END                             A 89

```

	C	INTEGRATION SIMPSON FOR N POINTS. N MUST BE EVEN.	B	1
	C	N MUST BE WITHIN 4 AND 100	B	2
0001		FUNCTION SIMP (DX,XX,N,J1)	B	3
0002		DIMENSION XX(100), DX(4)	B	4
0003		SIMP=XX(1)+XX(N)+4.*XX(N-1)	B	5
0004		J=(N-3)/2	B	6
0005		DO 1 I=1,J	B	7
0006	1	SIMP=SIMP+4.*XX(2*I)+2.*XX(2*I+1)	B	8
0007		SIMP=SIMP*DX(J1)/3.	B	9
0008		RETURN	B	10
0009		END	B	11

C		ACCELERATED 'REGULA FALSI'	C	1
C			C	2
C			C	3
0001		SUBROUTINE ZERO (A1,A2,F1,F2,A10,A20,DF1,DF2,RAD)	C	4
0002	1	RAD=A2-F2*(A2-A1)/(F2-F1)	C	5
0003		VARA=A2-F2*(A2-A1)/(F2-F1)	C	6
0004		HH=ABS(A20-A10)	C	7
0005		H1=ABS(VARA-A10)	C	8
0006		H2=ABS(VARA-A20)	C	9
0007		IF (H1-H2) 2,8,3	C	10
0008	2	TEMP=H2	C	11
0009		L=2	C	12
0010		GO TO 4	C	13
0011	3	TEMP=H1	C	14
0012		L=1	C	15
0013	4	IF (TEMP-HH) 8,5,5	C	16
0014	5	GO TO (6,7), L	C	17
0015	6	A2=A20	C	18
0016		F2=DF2	C	19
0017		GO TO 1	C	20
0018	7	A2=A10	C	21
0019		F2=DF1	C	22
0020		GO TO 1	C	23
0021	8	RETURN	C	24
0022		END	C	25

```

C      ENTRY OF DATA AND DIMENSIONS OF ARRAYS                                D    1
0001      DIMENSION NINT(4), RR(5), H(4), EPS(30), SCAO(3,3)                    D    2
0002      DIMENSION SCA(3,4), SR(3,4), D(3,4), XNF(3,3), XKF(3,3)              D    3
0003      DIMENSION SANT(300), DFMAX(3), FS(300), SUM(300), SDS(300), AIDD(3   D    4
      1,300)                                                                    D    5
0004      DIMENSION ALFA(300), BETA(300), S(300), FLUX(3,300), GX(100), FPR(   D    6
      13)                                                                        D    7
0005      COMMON AIDD,H,NINT                                                    D    8
0006      1 READ (5,142) OVREL,CONV,PREC,BH2,SCP,POW,NINT,ICP,ICC              D    9
0007      READ (5,143) ((D(I,K),SCA(I,K),SR(I,K),XNF(I,K),XKF(I,K),I=1,3),K=   D   10
      11,3)                                                                        D   11
0008      READ (5,144) (D(I,4),SCA(I,4),SR(I,4),I=1,3)                        D   12
0009      READ (5,145) (RR(K),K=2,5)                                           D   13
0010      READ (5,146) PP                                                       D   14
      C                                                                            D   15
      C      PRINTING OF DATA                                                  D   16
      C                                                                            D   17
0011      RR(1)=0.0                                                            D   18
0012      WRITE (6,147)                                                         D   19
0013      WRITE (6,148)                                                         D   20
0014      WRITE (6,149) (RR(L),L=1,5)                                          D   21
0015      WRITE (6,150) NINT,OVREL,PREC,CONV                                   D   22
0016      WRITE (6,151) BH2,SCP                                                D   23
0017      WRITE (6,152) ((SCA(I,K),I=1,3),K=1,4)                              D   24
0018      WRITE (6,153) ((D(I,K),I=1,3),K=1,4)                                D   25
0019      WRITE (6,154) ((SR(I,K),I=1,3),K=1,4)                              D   26
0020      WRITE (6,155) ((XNF(I,K),I=1,3),K=1,3)                              D   27
0021      WRITE (6,156) ((XKF(I,K),I=1,3),K=1,3)                              D   28
0022      WRITE (6,157) POW                                                     D   29
0023      WRITE (6,158)                                                         D   30
      C                                                                            D   31
      C      NUMBERING OF MESH-POINTS                                           D   32
      C                                                                            D   33
0024      PI=3.14159265                                                         D   34
0025      CF=1.6021E-13                                                         D   35
0026      TEMP=1.0                                                             D   36
0027      ITER=1                                                                D   37
0028      NIT=1                                                                D   38
0029      NINT(1)=NINT(1)+1                                                    D   39
0030      N1=NINT(1)                                                           D   40
0031      DO 2 K=2,4                                                           D   41
0032      2 NINT(K)=NINT(K)+NINT(K-1)                                          D   42
0033      N2=NINT(2)                                                           D   43
0034      N3=NINT(3)                                                           D   44
0035      N3M1=N3-1                                                            D   45
0036      N4=NINT(4)                                                           D   46
0037      N4M1=N4-1                                                            D   47
0038      XP=1.                                                                D   48
0039      JJ=1                                                                  D   49
0040      DO 3 K=1,4                                                           D   50
0041      DO 3 I=1,3                                                           D   51
0042      3 SCA(I,K)=SCA(I,K)+SR(I,K)+BH2*D(I,K)                              D   52
0043      DO 4 K=1,3                                                           D   53
0044      4 SCAO(3,K)=SCA(3,K)                                                D   54
0045      DO 5 N=1,N4M1                                                        D   55
0046      FS(N)=0.0                                                            D   56
0047      5 SUM(N)=0.0                                                         D   57
      C                                                                            D   58

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C      CALCULATION OF H(K)
C
0048      DO 8 K=1,4
0049      IF (K-1) 8,6,7
0050      6      TEMP=NINT(1)-1
0051      GO TO 8
0052      7      TEMP=NINT(K)-NINT(K-1)
0053      8      H(K)=(RR(K+1)-RR(K))/TEMP
C
C      POISON CONTROL BLOCK
C
0054      9      GO TO (10,10,10,11,12,13), ICP
0055      10     SCA(3,ICP)=SCA0(3,ICP)*XP
0056      GO TO 16
0057      11     KF=1
0058      KL=3
0059      GO TO 14
0060      12     KF=1
0061      KL=2
0062      GO TO 14
0063      13     KF=2
0064      KL=3
0065      14     DO 15 K=KF,KL
0066      15     SCA(3,K)=SCA0(3,K)*XP
0067      JJ=1
C
C      GENERATION OF THE INITIAL FAST SOURCE
C
0068      16     S(1)=1./(RR(4)*PR(4)+PI)
0069      ITC=1
0070      S(1)=S(1)
0071      DO 21 N=2,N4
0072      IF (N-N3) 18,19,17
0073      17     S(N)=0.0
0074      GO TO 21
0075      18     S(N)=S(1)
0076      GO TO 20
0077      19     S(N3)=S(1)*H(3)/(H(3)+H(4))
0078      20     S(ANT(N))=S(N)
0079      21     CONTINUE
0080      FMANT=1.0
C
C
C      CALCULATION OF FLUXES
C
0081      22     DO 58 I=1,3
0082      TEMP=H(1)*H(1)*SCA(I,1)+4.*D(I,1)
0083      ALFA(2)=4.*D(I,1)/TEMP
0084      BETA(2)=S(1)*H(1)*H(1)/TEMP
0085      R=0.0
0086      K=1
0087      DO 26 N=2,N4M1
0088      IF (N-NINT(K)) 23,24,23
0089      23     R=R+H(K)
0090      AN=(1.+H(K)/(2.*R))*D(I,K)/(H(K)*H(K))
0091      CN=(1.-H(K)/(2.*R))*D(I,K)/(H(K)*H(K))
0092      BN=AN+CN+SCA(I,K)
0093      GO TO 25

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0094      24      K=K+1                                D 117
0095              R=RR(K)                            D 118
0096              TEMP=(H(K-1)+H(K))*0.5             D 119
0097              AN=(1.+H(K)/(2.*R))*D(I,K)/(H(K)*TEMP) D 120
0098              CN=(1.-H(K-1)/(2.*R))*D(I,K-1)/(H(K-1)*TEMP) D 121
0099              BN=AN+CN+(SCA(I,K)*H(K)+SCA(I,K-1)*H(K-1))/(H(K)+H(K-1)) D 122
0100      25      TEMP=BN-CN*ALFA(N)                 D 123
0101              ALFA(N+1)=AN/TEMP                 D 124
0102      26      BETA(N+1)=(S(N)+CN*BETA(N))/TEMP   D 125
0103              BM=CN*(3.+H(4)/(1.066*D(I,4)))-AN D 126
0104              CM=4.*CN-BN                       D 127
0105              GO TO (28,27), ITER                D 128
0106      27      TEMP=FLUX(I,N4)                   D 129
0107      28      FLUX(I,N4)=(S(N4M1)+CM*BETA(N4))/(BM-CM*ALFA(N4)) D 130
0108              GO TO (30,29), ITER                D 131
0109      29      DFMAX(I)=ABS((TEMP-FLUX(I,N4))/TEMP) D 132
0110      30      DO 35 N=1,N4M1                     D 133
0111              L=N4-N                             D 134
0112              GO TO (32,31), ITER                D 135
0113      31      TEMP=FLUX(I,L)                     D 136
0114      32      FLUX(I,L)=ALFA(L+1)*FLUX(I,L+1)+BETA(L+1) D 137
0115              GO TO (35,33), ITER                D 138
0116      33      DFMAX(I)=ABS((TEMP-FLUX(I,L))/TEMP) D 139
0117              IF (DFMAX(I)-TEMP) 34,35,35       D 140
0118      34      DFMAX(I)=TEMP                       D 141
0119      35      CONTINUE                             D 142
C                                                D 143
C        CALCULATION OF SOURCE FOR THE FOLLOWING GROUP D 144
C                                                D 145
0120      K=1                                         D 146
0121      DO 58 N=1,N4M1                             D 147
0122      IF (N.GT.NINT(3)) GO TO 40                 D 148
0123      IF (N=NINT(K)) 36,40,36                    D 149
0124      36      GO TO (38,38,37), I                 D 150
0125      37      SDS(N)=0.0                           D 151
0126              GO TO 39                             D 152
0127      38      SDS(N)=SR(I,K)*FLUX(I,N)            D 153
0128      39      FS(N)=XNF(I,K)*FLUX(I,N)           D 154
0129              GO TO 53                             D 155
0130      40      K=K+1                                D 156
0131              IF (K-4) 45,41,50                   D 157
0132      41      GO TO (43,43,42), I                 D 158
0133      42      SDS(N)=0.0                           D 159
0134              TEMP1=0.0                             D 160
0135              GO TO 44                             D 161
0136      43      SDS(N)=SR(I,K)*FLUX(I,N)            D 162
0137              TEMP1=SR(I,K-1)*FLUX(I,N)           D 163
0138      44      TEMP2=XNF(I,K-1)*FLUX(I,N)         D 164
0139              FS(N)=0.0                             D 165
0140              GO TO 49                             D 166
0141      45      GO TO (47,47,46), I                 D 167
0142      46      SDS(N)=0.0                           D 168
0143              TEMP1=0.0                             D 169
0144              GO TO 48                             D 170
0145      47      SDS(N)=SR(I,K)*FLUX(I,N)            D 171
0146              TEMP1=SR(I,K-1)*FLUX(I,N)           D 172
0147      48      FS(N)=XNF(I,K)*FLUX(I,N)           D 173
0148              TEMP2=XNF(I,K-1)*FLUX(I,N)         D 174

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0149      49      TEMP=H(K)+H(K-1)                                D 175
0150      SDS(N)=(SDS(N)*H(K)+TEMP1*H(K-1))/TEMP              D 176
0151      FS(N)=(FS(N)*H(K)+TEMP2*H(K-1))/TEMP                D 177
0152      GO TO 53                                             D 178
0153      50      GO TO (52,52,51), I                          D 179
0154      51      SDS(N)=0.0                                    D 180
0155      GO TO 54                                             D 181
0156      52      SDS(N)=SR(I,4)*FLUX(I,N)                     D 182
0157      GO TO 54                                             D 183
0158      53      SUM(N)=SUM(N)+FS(N)                           D 184
0159      54      GO TO (56,56,55), I                          D 185
0160      55      IF (N.GT.NINT(3)) GO TO 57                   D 186
0161      S(N)=SUM(N)                                          D 187
0162      SUM(N)=0.0                                           D 188
0163      GO TO 58                                             D 189
0164      56      S(N)=SDS(N)                                    D 190
0165      GO TO 58                                             D 191
0166      57      S(N)=0.0                                       D 192
0167      58      CONTINUE                                       D 193
C                                                D 194
C      INTEGRATION OF THE SOURCE                               D 195
C                                                D 196
0168      FMULT=S(N3)*RR(4)*H(4)/3.                            D 197
0169      DO 63 K=1,3                                          D 198
0170      GO TO (59,60,60), K                                  D 199
0171      59      L=0                                           D 200
0172      GO TO 61                                             D 201
0173      60      L=NINT(K-1)-1                                   D 202
0174      61      M=NINT(K)-L                                   D 203
0175      R=RR(K)                                             D 204
0176      DO 62 N=1,M                                         D 205
0177      ITEMP=N+L                                           D 206
0178      GX(N)=S(ITEMP)*R                                     D 207
0179      62      R=R+H(K)                                       D 208
0180      63      FMULT=FMULT+SIMP(H,GX,M,K)                   D 209
0181      FMULT=FMULT*2.*PI                                    D 210
0182      WRITE (6,159) FMULT                                  D 211
0183      IF (ABS(FMULT-FMANT)/FMULT-PREC/5.) 85,64,64       D 212
C                                                D 213
C      PREPARATION OF ITERATIONS                               D 214
C                                                D 215
0184      64      RPSP=0.0                                       D 216
0185      DO 66 N=1,N3                                         D 217
0186      S(N)=S(N)/FMULT                                       D 218
0187      EMAX=ABS((S(N)-SANT(N))/SANT(N))                     D 219
0188      IF (EMAX-RPSP) 66,66,65                               D 220
0189      65      RPSP=EMAX                                       D 221
0190      66      CONTINUE                                       D 222
0191      EPS(JJ)=RPSP                                          D 223
0192      IF (JJ-20) 72,67,67                                   D 224
0193      67      IF (ABS(EPS(10)/EPS(20))-1.5) 68,71,71      D 225
0194      68      IF (ABS(EPS(20)/EPS(19))*(1.+ABS(EPS(19))))-1.) 71,69,69 D 226
0195      69      GO TO (70,71), ITER                            D 227
0196      70      OVREL=OVREL-0.1                                 D 228
0197      71      JJ=1                                           D 229
0198      72      DO 84 N=1,N3                                     D 230
0199      Q=S(N)/SANT(N)-1.                                     D 231
0200      IF (Q) 73,74,74                                       D 232

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0201      73      K1=1                                D 233
0202      GO TO 75                                    D 234
0203      74      K1=2                                D 235
0204      75      Q=ABS(Q)*GVREL                       D 236
0205      IF (Q-0.1) 77,76,76                        D 237
0206      76      Q=1.0/EXP(Q)                        D 238
0207      GO TO 80                                    D 239
0208      77      IF (Q-.001) 79,78,78                D 240
0209      78      Q=(1.-Q/2.)/(1.+Q/2.)                D 241
0210      GO TO 80                                    D 242
0211      79      Q=1.-Q                                D 243
0212      80      GO TO (81,82), K1                    D 244
0213      81      S(N)=SANT(N)*Q                       D 245
0214      GO TO 83                                    D 246
0215      82      S(N)=SANT(N)*(2.-Q)                 D 247
0216      83      SANT(N)=S(N)                         D 248
0217      84      CONTINUE                             D 249
0218      FMANT=FMULT                                 D 250
0219      JJ=JJ+1                                     D 251
0220      ITC=ITC+1                                   D 252
0221      IF (ITC.GT.400) GO TO 140                    D 253
0222      GO TO 22                                     D 254
0223      85      GO TO (89,86), ITER                   D 255
0224      86      DFMM=0.0                              D 256
0225      DO 88 I=1,3                                  D 257
0226      IF (DFMAX(I)-DFMM) 88,88,87                 D 258
0227      87      DFMM=DFMAX(I)                        D 259
0228      88      CCNTINUE                             D 260
0229      IF (DFMM-CONV) 89,64,64                     D 261
0230      89      FME=1.                                D 262
0231      DFML=FMULT-FME                               D 263
0232      WRITE (6,160) XP,FMULT                       D 264
0233      IF (XP.EQ.1.0) EIGEN=FMULT                  D 265
0234      IF (ABS(DFML)-PREC) 104,90,90                D 266
0235      90      GO TO (91,92,99,103), NIT             D 267
0236      91      DFMI=DFML                             D 268
0237      NIT=2                                         D 269
0238      GO TO 93                                     D 270
0239      92      IF (DFML*DFMI) 95,104,93              D 271
0240      93      IF (DFML) 94,104,94                  D 272
0241      94      X1=XP                                  D 273
0242      XP=XP+PP*DFML/ABS(DFML)
0243      DFMI=DFML
0244      GO TO 9
0245      95      IF (DFML) 96,104,97                    D 275
0246      96      DF2=DFMI                               D 276
0247      DF1=DFML                                     D 277
0248      A20=X1                                       D 278
0249      A10=XP                                       D 279
0250      XP=X1-DFMI*(X1-XP)/(DFMI-DFML)              D 280
0251      GO TO 98                                     D 281
0252      97      DF2=DFML                               D 282
0253      DF1=DFMI                                     D 283
0254      A20=XP                                       D 284
0255      A10=X1                                       D 285
0256      XP=XP-DFML*(XP-X1)/(DFML-DFMI)              D 286
0257      98      NIT=3                                 D 287
0258      X1=XP                                       D 288

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0259          DFM1=DFMUL                                D 290
0260          GO TO 9                                    D 291
0261          99   NIT=4                                  D 292
0262          DFM1=DFMUL                                D 293
0263          X1=XP                                      D 294
0264          IF (DFMUL) 100,104,101                    D 295
0265          100  XP=X1-DFM1*(A20-X1)/(DF2-DFM1)        D 296
0266          GO TO 102                                  D 297
0267          101  XP=X1-DFM1*(X1-A10)/(DFM1-DF1)        D 298
0268          102  XANT=XP                                D 299
0269          GO TO 9                                    D 300
0270          103  CALL ZERO (X1,XANT,DFM1,DFMUL,A10,A20,DF1,DF2,XP) D 301
0271          X1=XANT                                    D 302
0272          XANT=XI                                    D 303
0273          DFM1=DFMUL                                D 304
0274          GO TO 9                                    D 305
0275          104  GO TO (105,106), ITER                  D 306
0276          105  ITER=2                                D 307
0277          WRITE (6,161) ITER                         D 308
0278          GO TO 64                                    D 309
C                                                     D 310
C                                                     D 311
C   PREPARATION OF OUTPUT                             D 312
C   CALCULATION OF POWER NORMALIZED FLUXES           D 313
C                                                     D 313
0279          106  FP=0.0                                 D 314
0280          DO 107 N=1,N3                               D 315
0281          107  SUM(N)=0.0                             D 316
0282          K=1                                         D 317
0283          DO 110 I=1,3                               D 318
0284          DO 110 N=1,N3M1                             D 319
0285          IF (N=NINT(K)) 108,109,108                 D 320
0286          108  FS(N)=XKF(I,K)*FLUX(I,N)               D 321
0287          GO TO 110                                   D 322
0288          109  K=K+1                                   D 323
0289          TEMP=XKF(I,K-1)*FLUX(I,N)                   D 324
0290          FS(N)=XKF(I,K)*FLUX(I,N)                   D 325
0291          FS(N)=(FS(N)*H(K)+TEMP*H(K-1))/(H(K)+H(K-1)) D 326
0292          110  SUM(N)=SUM(N)+FS(N)                     D 327
0293          TEMP=H(3)/(H(3)+H(4))                       D 328
0294          DO 111 I=1,3                               D 329
0295          111  SUM(N3)=SUM(N3)+XKF(I,3)*FLUX(I,N3)*TEMP D 330
0296          DO 119 K=1,3                                 D 331
0297          GO TO (112,113,113), K                       D 332
0298          112  L=0                                     D 333
0299          GO TO 114                                    D 334
0300          113  L=NINT(K-1)-1                           D 335
0301          114  M=NINT(K)-L                             D 336
0302          R=RR(K)                                       D 337
0303          DO 115 N=1,M                                 D 338
0304          ITEMP=N+L                                     D 339
0305          GX(N)=SUM(ITEMP)*R                           D 340
0306          115  R=R+H(K)                                 D 341
0307          FP=FP+5*IMP(H,GX,M,K)                       D 342
0308          IF (K.EQ.3) FP=FP+SUM(N3)*RR(4)*H(4)/3.    D 343
0309          GO TO (116,117,118), K                       D 344
0310          116  FPR(K)=FP                                 D 345
0311          GO TO 119                                    D 346
0312          117  FPR(K)=(FP-FPR(K-1))                    D 347

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0313          GO TO 119
0314          118  FPR(K)=(FP-FPR(K-1)-FPR(K-2))
0315          119  CONTINUE
0316          FP=0.0
0317          DO 120 K=1,3
0318          FPR(K)=2.*PI*FPR(K)
0319          120  FP=FP+FPR(K)
0320          GAM=POW/FP
0321          TEMP=PI/SQRT(BH2)
0322          GAM=GAM/TEMP
0323          WRITE (6,162)
0324          WRITE (6,163) (RR(L),L=1,5)
0325          WRITE (6,164) EIGEN
0326          WRITE (6,165)
0327          GO TO (121,121,121,122,123,124), ICP
0328          121  WRITE (6,166) ICP
0329          GO TO 125
0330          122  WRITE (6,167)
0331          GO TO 125
0332          123  WRITE (6,168)
0333          GO TO 125
0334          124  WRITE (6,169)
0335          125  DO 126 K=1,3
0336          126  SCA(3,K)=SCA(3,K)-D(3,K)*BH2
0337          GO TO (127,128,129,127,127,128), ICP
0338          127  K=1
0339          GO TO 130
0340          128  K=2
0341          GO TO 130
0342          129  K=3
0343          130  TEMP=SCA(3,K)-SCA0(3,K)
0344          WRITE (6,170) TEMP
0345          TEMP=TEMP/SCP*1.E+24
0346          WRITE (6,171) TEMP
0347          WRITE (6,172)
0348          WRITE (6,173) (FLUX(1,N),N=1,N4)
0349          WRITE (6,174) (FLUX(2,N),N=1,N4)
0350          WRITE (6,175) (FLUX(3,N),N=1,N4)

C
C  CALCULATION OF POWER NORMALIZED FLUXES
C
0351          DO 131 I=1,3
0352          DO 131 N=1,N4
0353          FLUX(I,N)=FLUX(I,N)*GAM
0354          131  AIDD(I,N)=FLUX(I,N)
0355          WRITE (6,176)
0356          WRITE (6,177) (AIDD(1,N),N=1,N4)
0357          WRITE (6,178) (AIDD(2,N),N=1,N4)
0358          WRITE (6,179) (AIDD(3,N),N=1,N4)
0359          DO 132 LI=1,3
0360          LP=LI
0361          CALL PLOT (LI,N4,LP)
0362          132  CONTINUE
0363          WRITE (6,180)
0364          DO 133 I=1,3
0365          TEMP=FLUX(I,1)
0366          DO 133 N=1,N4
0367          133  AIDD(I,N)=FLUX(I,N)/TEMP

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0368 WRITE (6,181) (AIDD(1,N),N=1,N4)
0369 WRITE (6,182) (AIDD(2,N),N=1,N4)
0370 WRITE (6,183) (AIDD(3,N),N=1,N4)
0371 DO 134 LI=1,3
0372 LP=LI+3
0373 CALL PLOT (LI,N4,LP)
0374 134 CONTINUE
0375 DO 135 I=1,2
0376 DO 135 N=1,N4
0377 135 AIDD(I,N)=FLUX(I,N)/FLUX(3,N)
0378 WRITE (6,184)
0379 WRITE (6,185) (AIDD(1,N),N=1,N4)
0380 WRITE (6,186) (AIDD(2,N),N=1,N4)
0381 DO 136 LI=1,2
0382 LP=6+LI
0383 CALL PLOT (LI,N4,LP)
0384 136 CONTINUE
0385 DO 137 N=1,N4
0386 137 AIDD(1,N)=FLUX(1,N)/FLUX(2,N)
0387 WRITE (6,187) (AIDD(1,N),N=1,N4)
0388 CALL PLOT (1,N4,9)

C
C CALCULATION OF POWER GENERATED IN EACH REGION
C
0389 DO 138 K=1,3
0390 TEMP=PI/SQRT(BH2)
0391 FPR(K)=FPR(K)*TEMP*GAM
0392 138 CONTINUE
0393 WRITE (6,188) (FPR(K),K=1,3)
0394 DO 139 K=1,3
0395 FPR(K)=FPR(K)/TEMP
0396 TEMP=PI*(RR(K+1)*RR(K+1)-RR(K)*RR(K))
0397 FPR(K)=FPR(K)/TEMP
0398 139 CONTINUE
0399 WRITE (6,189) (FPR(K),K=1,3)
0400 WRITE (6,190) OVREL
0401 IF (ICC.GT.0) GO TO 1
0402 GO TO 141
0403 140 WRITE (6,191)
0404 141 STCP

C
0405 142 FORMAT (6E10.0,6(1X,12))
0406 143 FORMAT (5E10.0)
0407 144 FORMAT (3E10.0)
0408 145 FORMAT (4E10.0)
0409 146 FORMAT (F5.3)
0410 147 FORMAT (1H1)
0411 148 FORMAT (16H PROGRAM WAN DIC,///,26H REGIONS IN ORDER 1,2,3,4,/)
0412 149 FORMAT (17H RADII OF REGIONS,/,2X,8HREGION 1,4X,8HREGION 2,4X,8HREGION 3,4X,8HREGION 4,4X,8HREGION 5,/,2X,5(F8.2,4X))
0413 150 FORMAT (7H NINT =,4(12,1X),/,7H OVREL=,F6.2,6X,6H PREC=,E9.2,6X,6H 1 CONV=,E9.2)
0414 151 FORMAT (/,6H BH2 =,E12.5,6X,28H POISON MICRO CROSS-SECTION=,1X,E12.5,/)
0415 152 FORMAT (26H ABSORPTION CROSS-SECTIONS,/,5X,7HGROUP 1,7X,7HGROUP 2 1,7X,7HGROUP 3,/,3(2X,E12.5),/)
0416 153 FORMAT (/,23H DIFFUSION COEFFICIENTS,/,3(2X,E12.5),/)
0417 154 FORMAT (/,26H TRANSFER CROSS-SECTIONS ,/,3(2X,E12.5),/)

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0418      155  FORMAT (//,23H NU-FISSION                ,//,3(2X,E12.5),/)          D 464
0419      156  FORMAT (//,14H KAPPA-FISSION,//,3(2X,E12.5),/)          D 465
0420      157  FORMAT (13H PCWER (W) = ,E12.5)                          D 466
0421      158  FORMAT (1H1)                                              D 467
0422      159  FCRMAT (1X,4H K=,F10.6)                                    D 468
0423      160  FORMAT (//,5H XP= ,E12.5,5X,15H EIGENVALUE K =,E12.5)    D 469
0424      161  FORMAT (7H ITER= ,I2)                                       D 470
0425      162  FORMAT (1H1,8H RESULTS,///)                                D 471
0426      163  FORMAT (17H RADII OF REGIONS,//,2X,8HREGION 1,4X,8HREGION 2,4X,8HR
          1EGION 3,4X,8HREGION 4,4X,8HREGION 5,//,2X,5(F8.2,4X))          D 473
0427      164  FORMAT (//,41H EIGENVALUE OF THE UNPOISONED REACTOR K =,E12.5) D 474
0428      165  FORMAT (//,49H CONTROL POISON WAS USED IN THE FOLLOWING REGIONS,/) D 475
0429      166  FORMAT (9H REGION ,I2)                                       D 476
0430      167  FCRMAT (21H ALL FISSION REGIONS )                          D 477
0431      168  FORMAT (17H REGIONS 1 AND 2 )                               D 478
0432      169  FORMAT (17H REGIONS 2 AND 3 )                               D 479
0433      170  FORMAT (//,36H MACRO CROSS-SECTION OF THE POISON= ,E12.5) D 480
0434      171  FORMAT (//,24H POISON CONCENTRATION = ,E12.5)             D 481
0435      172  FORMAT (////,28H FLUXES NORMALIZED TO SOURCE)             D 482
0436      173  FORMAT (//,19H FAST FLUX-GROUP 1 ,/,5(2X,E12.5))          D 483
0437      174  FORMAT (//,24H RESONANCE FLUX-GROUP 2 ,/,5(2X,E12.5))    D 484
0438      175  FORMAT (//,19H SLOW FLUX-GROUP 3 ,/,5(2X,E12.5))        D 485
0439      176  FORMAT (////,28H FLUXES NORMALIZED TO PCWER )            D 486
0440      177  FORMAT (//,19H FAST FLUX-GROUP 1 ,/,5(2X,E12.5))          D 487
0441      178  FORMAT (//,24H RESONANCE FLUX-GROUP 2 ,/,5(2X,E12.5))    D 488
0442      179  FORMAT (//,19H SLOW FLUX-GROUP 3 ,/,5(2X,E12.5))        D 489
0443      180  FCRMAT (////,16H RELATIVE FLUXES)                          D 490
0444      181  FORMAT (//,9H GROUP 1 ,/,5(2X,E12.5))                     D 491
0445      182  FORMAT (//,9H GROUP 2 ,/,5(2X,E12.5))                     D 492
0446      183  FORMAT (//,9H GROUP 3 ,/,5(2X,E12.5))                     D 493
0447      184  FORMAT (////,35H RATIOS OF FLUXES TO THE SLOW FLUX )      D 494
0448      185  FORMAT (//,19H GROUP 1 / GROUP 3 ,/,5(2X,E12.5))          D 495
0449      186  FORMAT (//,19H GROUP 2 / GROUP 3 ,/,5(2X,E12.5))          D 496
0450      187  FORMAT (//,19H GROUP 1 / GROUP 2 ,//,5(2X,E12.5))        D 497
0451      188  FORMAT (//,31H POWER OUTPUT IN REGIONS 1,2,3,//,2X,3(E12.5,5X)) D 498
0452      189  FORMAT (//,42H POWER DENSITY IN REGIONS 1,2,3 (KW/LITER),//,2X,3(E
          112.5,5X))                                                       D 500
0453      190  FORMAT (//,33H COMPUTED OVERRELAXATION FACTOR= ,F6.3)    D 501
0454      191  FORMAT (22H # OF ITERATIONS > 400)                        D 502
0455      END                                                                D 503

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PROGRAM WAN DIC

REGIONS IN ORDER 1,2,3,4,

RADII OF REGIONS

REGION 1	REGION 2	REGION 3	REGION 4	REGION 5
0.0	87.70	124.00	152.00	159.00

NINT = 30 20 20 4
OVREL= 1.90 PREC= 0.10E-03 CONV= 0.10E-01

BH2 = 0.73800E-04 POISON MICRO CROSS-SECTION= 0.21090E 04

ABSORPTION CROSS-SECTIONS

GROUP 1	GROUP 2	GROUP 3
0.35410E-02	0.25680E-01	0.13870E 00
0.35410E-02	0.25680E-01	0.13870E 00
0.35410E-02	0.25680E-01	0.13870E 00
0.36220E-03	0.57220E-03	0.84730E-02

DIFFUSION COEFFICIENTS

0.17070E 01	0.85360E 00	0.39510E 00
0.17070E 01	0.85360E 00	0.39510E 00
0.17070E 01	0.85360E 00	0.39510E 00
0.19300E 01	0.91730E 00	0.31470E 00

TRANSFER CROSS-SECTIONS

0.61130E-01	0.49440E-01	0.0
0.61130E-01	0.49440E-01	0.0
0.61130E-01	0.49440E-01	0.0
0.26280E-01	0.28110E-01	0.0

NU-FISSION

0.42920E-02	0.15060E-01	0.16400E 00
0.42920E-02	0.15060E-01	0.16400E 00
0.42920E-02	0.15060E-01	0.16400E 00

KAPPA-FISSION

0.93280E-13	0.19010E-12	0.21290E-11
0.93280E-13	0.19010E-12	0.21290E-11
0.93280E-13	0.19010E-12	0.21290E-11

POWER (W) = 0.24410E 10

RESULTS

RAOII OF REGIONS

REGION 1	REGION 2	REGION 3	REGION 4	REGION 5
0.0	87.70	124.00	152.00	159.00

EIGENVALUE OF THE UNPOISONED REACTOR K = 0.97971E 00

CONTROL POISON WAS USED IN THE FOLLOWING REGIONS

ALL FISSION REGIONS

MACRO CROSS-SECTION OF THE POISON= -0.37987E-02

POISON CONCENTRATION = -0.18012E 19

FLUXES NORMALIZED TO SOURCE

FAST FLUX-GROUP 1

0.42922E-03	0.42904E-03	0.42848E-03	0.42755E-03	0.42625E-03
0.42457E-03	0.42253E-03	0.42012E-03	0.41734E-03	0.41420E-03
0.41070E-03	0.40683E-03	0.40261E-03	0.39804E-03	0.39312E-03
0.38785E-03	0.38224E-03	0.37629E-03	0.37002E-03	0.36342E-03
0.35650E-03	0.34927E-03	0.34173E-03	0.33390E-03	0.32578E-03
0.31737E-03	0.30870E-03	0.29977E-03	0.29059E-03	0.28117E-03
0.27153E-03	0.26543E-03	0.25925E-03	0.25300E-03	0.24667E-03
0.24027E-03	0.23381E-03	0.22728E-03	0.22069E-03	0.21404E-03
0.20734E-03	0.20059E-03	0.19379E-03	0.18696E-03	0.18008E-03
0.17317E-03	0.16623E-03	0.15926E-03	0.15228E-03	0.14527E-03
0.13825E-03	0.13282E-03	0.12740E-03	0.12197E-03	0.11654E-03
0.11111E-03	0.10569E-03	0.10027E-03	0.94856E-04	0.89454E-04
0.84063E-04	0.78686E-04	0.73325E-04	0.67980E-04	0.62653E-04
0.57341E-04	0.52038E-04	0.46722E-04	0.41343E-04	0.35784E-04
0.29774E-04	0.23213E-04	0.17707E-04	0.13012E-04	0.89179E-05

RESONANCE FLUX-GROUP 2

0.34819E-03	0.34804E-03	0.34759E-03	0.34683E-03	0.34577E-03
0.34441E-03	0.34275E-03	0.34080E-03	0.33854E-03	0.33599E-03
0.33314E-03	0.33001E-03	0.32658E-03	0.32287E-03	0.31887E-03
0.31459E-03	0.31004E-03	0.30521E-03	0.30012E-03	0.29476E-03
0.28914E-03	0.28327E-03	0.27716E-03	0.27080E-03	0.26421E-03
0.25739E-03	0.25036E-03	0.24311E-03	0.23566E-03	0.22802E-03
0.22019E-03	0.21525E-03	0.21023E-03	0.20516E-03	0.20003E-03
0.19484E-03	0.18959E-03	0.18429E-03	0.17895E-03	0.17356E-03
0.16812E-03	0.16265E-03	0.15714E-03	0.15159E-03	0.14602E-03
0.14041E-03	0.13478E-03	0.12913E-03	0.12347E-03	0.11778E-03
0.11209E-03	0.10769E-03	0.10329E-03	0.98890E-04	0.94487E-04
0.90086E-04	0.85687E-04	0.81293E-04	0.76904E-04	0.72522E-04
0.68149E-04	0.63786E-04	0.59433E-04	0.55091E-04	0.50761E-04
0.46441E-04	0.42132E-04	0.37833E-04	0.33548E-04	0.29291E-04
0.25108E-04	0.20280E-04	0.15418E-04	0.10537E-04	0.55811E-05

SLOW FLUX-GROUP 3

0.12748E-03	0.12742E-03	0.12726E-03	0.12698E-03	0.12659E-03
0.12609E-03	0.12549E-03	0.12477E-03	0.12394E-03	0.12301E-03
0.12197E-03	0.12082E-03	0.11956E-03	0.11820E-03	0.11674E-03
0.11517E-03	0.11351E-03	0.11174E-03	0.10987E-03	0.10791E-03
0.10586E-03	0.10371E-03	0.10147E-03	0.99139E-04	0.96725E-04
0.94229E-04	0.91653E-04	0.89000E-04	0.86272E-04	0.83474E-04
0.80609E-04	0.78797E-04	0.76562E-04	0.75105E-04	0.73225E-04
0.71325E-04	0.69405E-04	0.67466E-04	0.65509E-04	0.63535E-04
0.61545E-04	0.59541E-04	0.57524E-04	0.55494E-04	0.53452E-04
0.51401E-04	0.49340E-04	0.47272E-04	0.45197E-04	0.43117E-04
0.41033E-04	0.39423E-04	0.37812E-04	0.36200E-04	0.34588E-04
0.32977E-04	0.31367E-04	0.29758E-04	0.28152E-04	0.26548E-04
0.24948E-04	0.23353E-04	0.21764E-04	0.20186E-04	0.18625E-04
0.17100E-04	0.15647E-04	0.14350E-04	0.13391E-04	0.13176E-04
0.14593E-04	0.16797E-04	0.14841E-04	0.99331E-05	0.30294E-05

FLUXES NORMALIZED TO POWER

FAST FLUX-GROUP 1

0.21237E 15	0.21228E 15	0.21200E 15	0.21154E 15	0.21090E 15
0.21007E 15	0.20906E 15	0.20787E 15	0.20649E 15	0.20494E 15
0.20320E 15	0.20129E 15	0.19920E 15	0.19694E 15	0.19451E 15
0.19190E 15	0.18912E 15	0.18618E 15	0.18308E 15	0.17981E 15
0.17639E 15	0.17281E 15	0.16908E 15	0.16520E 15	0.16119E 15
0.15703E 15	0.15274E 15	0.14832E 15	0.14378E 15	0.13912E 15
0.13435E 15	0.13133E 15	0.12827E 15	0.12518E 15	0.12205E 15
0.11888E 15	0.11568E 15	0.11245E 15	0.10919E 15	0.10590E 15
0.10259E 15	0.99247E 14	0.95885E 14	0.92502E 14	0.89100E 14
0.85681E 14	0.82248E 14	0.78801E 14	0.75343E 14	0.71876E 14
0.68402E 14	0.65719E 14	0.63034E 14	0.60347E 14	0.57661E 14
0.54975E 14	0.52291E 14	0.49610E 14	0.46933E 14	0.44260E 14
0.41593E 14	0.38932E 14	0.36279E 14	0.33635E 14	0.30999E 14
0.28371E 14	0.25747E 14	0.23117E 14	0.20456E 14	0.17705E 14
0.14731E 14	0.11485E 14	0.87612E 13	0.64379E 13	0.44124E 13

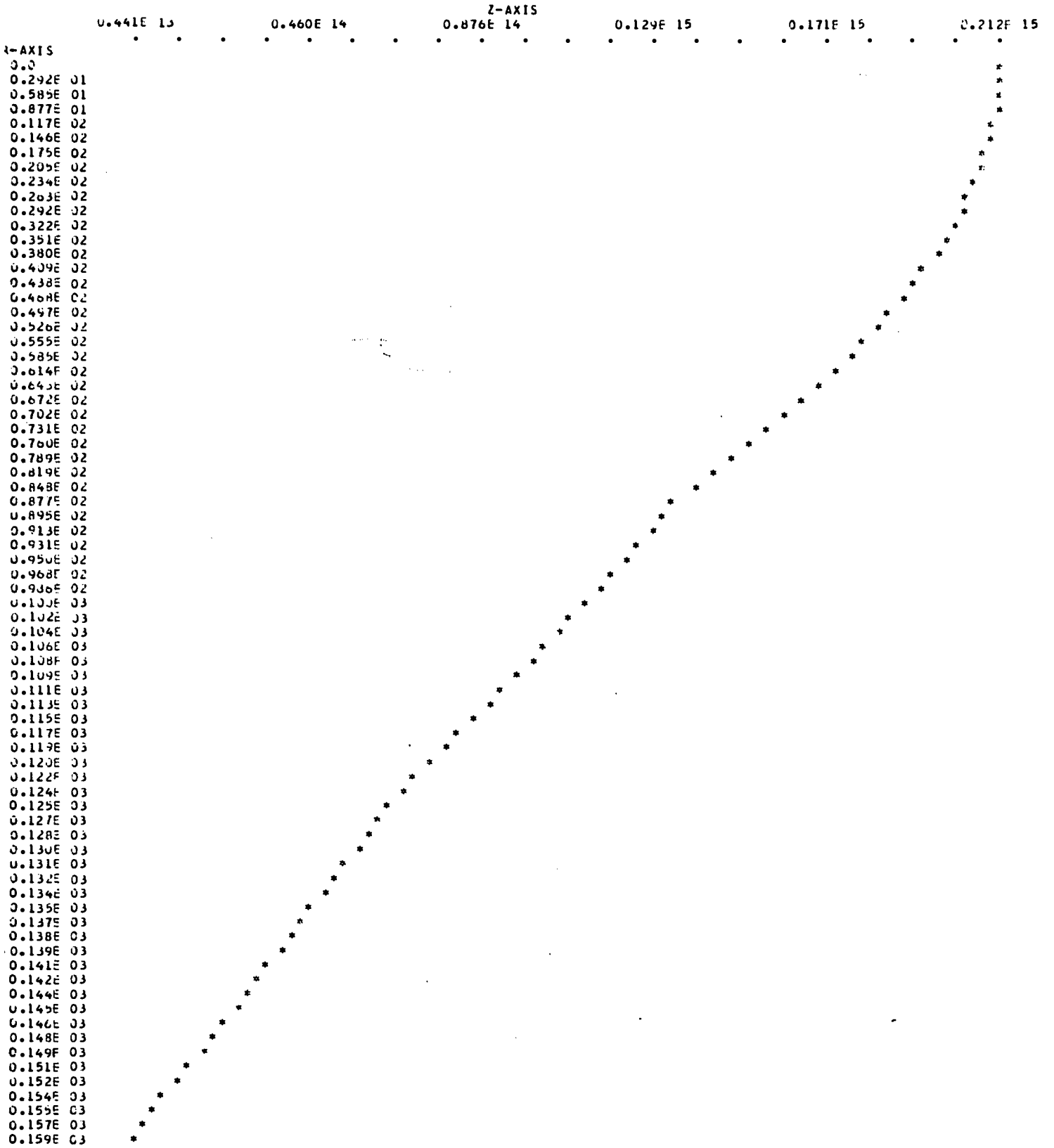
RESONANCE FLUX-GROUP 2

0.17228E 15	0.17220E 15	0.17198E 15	0.17160E 15	0.17108E 15
0.17041E 15	0.16959E 15	0.16862E 15	0.16750E 15	0.16624E 15
0.16483E 15	0.16328E 15	0.16159E 15	0.15975E 15	0.15777E 15
0.15565E 15	0.15340E 15	0.15101E 15	0.14849E 15	0.14584E 15
0.14306E 15	0.14016E 15	0.13713E 15	0.13399E 15	0.13073E 15
0.12735E 15	0.12387E 15	0.12029E 15	0.11660E 15	0.11282E 15
0.10895E 15	0.10650E 15	0.10402E 15	0.10151E 15	0.98969E 14
0.96400E 14	0.93805E 14	0.91185E 14	0.88540E 14	0.85872E 14
0.83184E 14	0.80475E 14	0.77748E 14	0.75005E 14	0.72246E 14
0.69473E 14	0.66688E 14	0.63893E 14	0.61089E 14	0.58277E 14
0.55460E 14	0.53284E 14	0.51107E 14	0.48929E 14	0.46750E 14
0.44572E 14	0.42396E 14	0.40222E 14	0.38050E 14	0.35882E 14
0.33719E 14	0.31560E 14	0.29406E 14	0.27258E 14	0.25115E 14
0.22978E 14	0.20846E 14	0.18719E 14	0.16599E 14	0.14403E 14
0.12423E 14	0.10034E 14	0.76287E 13	0.52137E 13	0.27600E 13

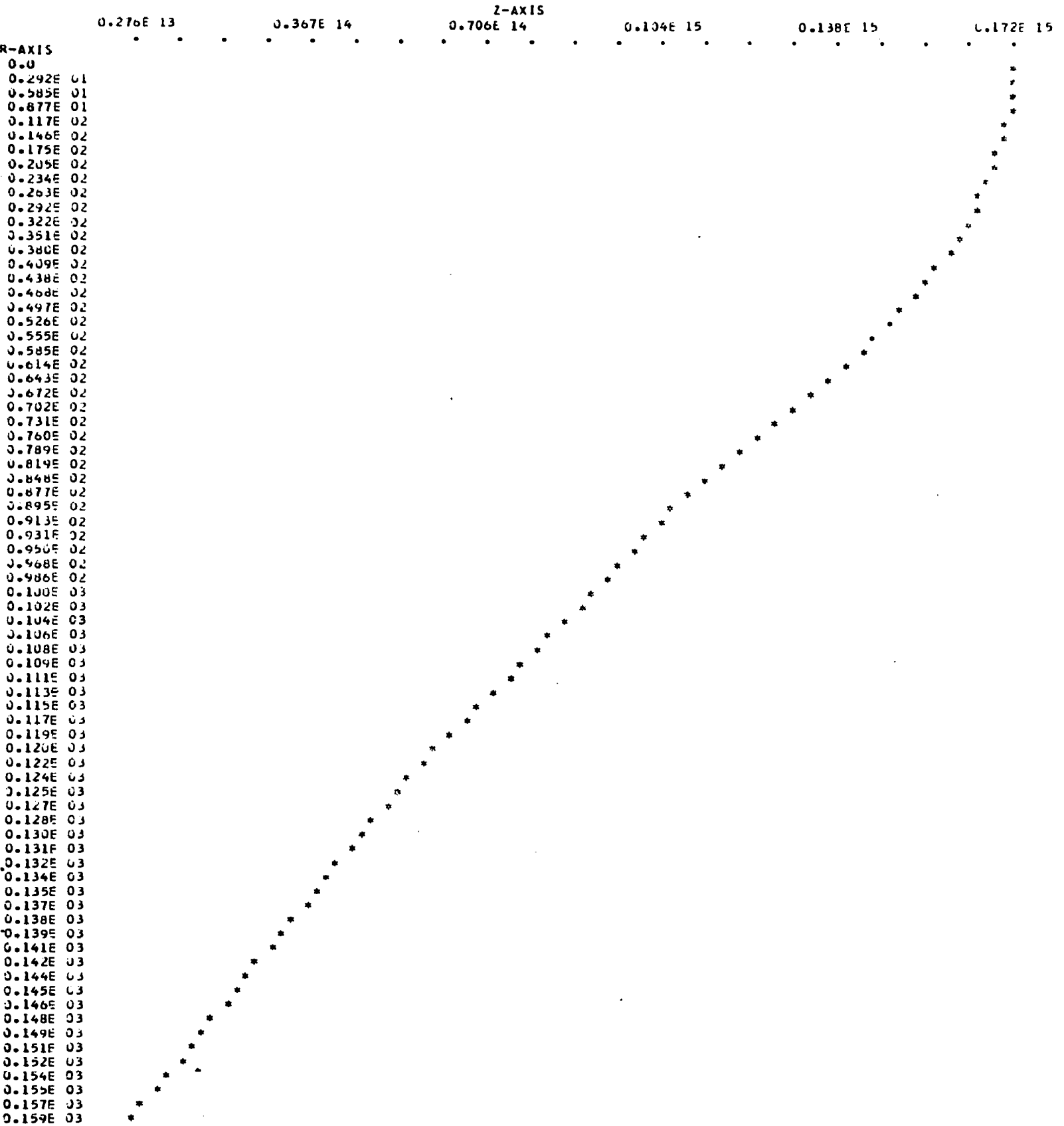
SLOW FLUX-GROUP 3

0.63073E 14	0.63046E 14	0.62963E 14	0.62826E 14	0.62635E 14
0.62388E 14	0.62088E 14	0.61733E 14	0.61324E 14	0.60862E 14
0.60347E 14	0.59778E 14	0.59158E 14	0.58485E 14	0.57761E 14
0.56986E 14	0.56161E 14	0.55286E 14	0.54363E 14	0.53392E 14
0.52375E 14	0.51311E 14	0.50203E 14	0.49052E 14	0.47858E 14
0.46623E 14	0.45348E 14	0.44035E 14	0.42686E 14	0.41301E 14
0.39883E 14	0.38987E 14	0.38079E 14	0.37160E 14	0.36230E 14
0.35290E 14	0.34340E 14	0.33380E 14	0.32412E 14	0.31436E 14
0.30451E 14	0.29460E 14	0.28461E 14	0.27457E 14	0.26447E 14
0.25432E 14	0.24412E 14	0.23389E 14	0.22363E 14	0.21333E 14
0.20302E 14	0.19506E 14	0.18708E 14	0.17911E 14	0.17114E 14
0.16316E 14	0.15520E 14	0.14724E 14	0.13929E 14	0.13135E 14
0.12344E 14	0.11554E 14	0.10768E 14	0.99874E 13	0.92153E 13
0.84606E 13	0.77418E 13	0.70999E 13	0.66258E 13	0.65193E 13
0.72204E 13	0.83110E 13	0.73431E 13	0.49147E 13	0.14989E 13

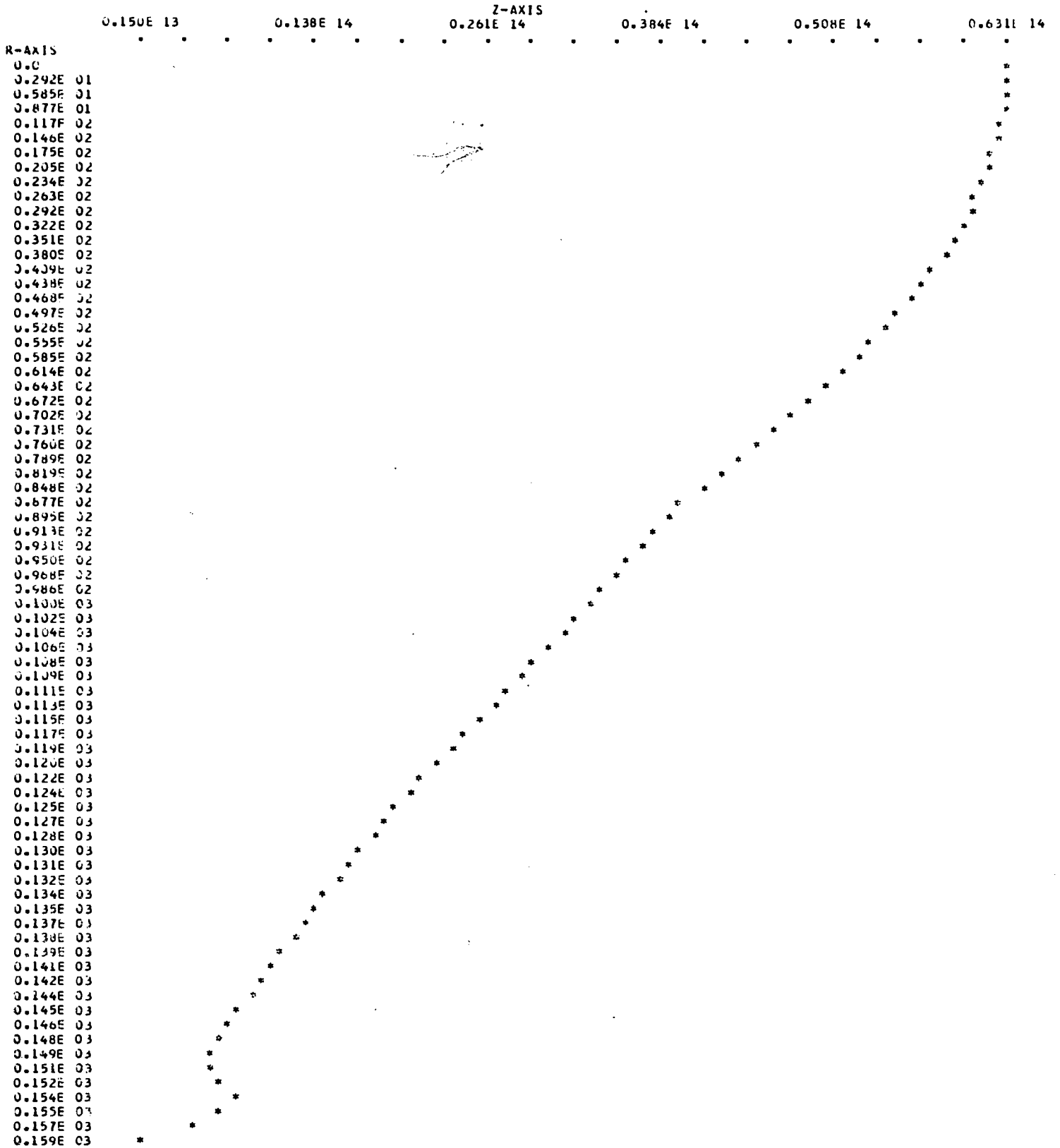
PLOT OF FAST FLUX - GROUP 1



PLOT OF RESONANCE FLUX -GROUP 2



PLOT OF SLOW FLUX - GROUP 3



RELATIVE FLUXES

GROUP 1

0.1000E 01	0.99957E 00	0.99826E 00	0.99609E 00	0.99306E 00
0.98916E 00	0.98440E 00	0.97879E 00	0.97232E 00	0.96500E 00
0.95683E 00	0.94783E 00	0.93800E 00	0.92735E 00	0.91588E 00
0.90360E 00	0.89053E 00	0.87668E 00	0.86206E 00	0.84668E 00
0.83056E 00	0.81371E 00	0.79616E 00	0.77791E 00	0.75899E 00
0.73942E 00	0.71921E 00	0.69841E 00	0.67702E 00	0.65507E 00
0.63260E 00	0.61839E 00	0.60400E 00	0.58943E 00	0.57469E 00
0.55978E 00	0.54472E 00	0.52950E 00	0.51415E 00	0.49866E 00
0.48305E 00	0.46733E 00	0.45150E 00	0.43557E 00	0.41955E 00
0.40345E 00	0.38728E 00	0.37105E 00	0.35477E 00	0.33845E 00
0.32209E 00	0.30945E 00	0.29681E 00	0.28416E 00	0.27151E 00
0.25886E 00	0.24623E 00	0.23360E 00	0.22099E 00	0.20841E 00
0.19585E 00	0.18332E 00	0.17083E 00	0.15838E 00	0.14597E 00
0.13359E 00	0.12124E 00	0.10885E 00	0.96321E-01	0.83369E-01
0.69366E-01	0.54081E-01	0.41254E-01	0.30314E-01	0.20777E-01

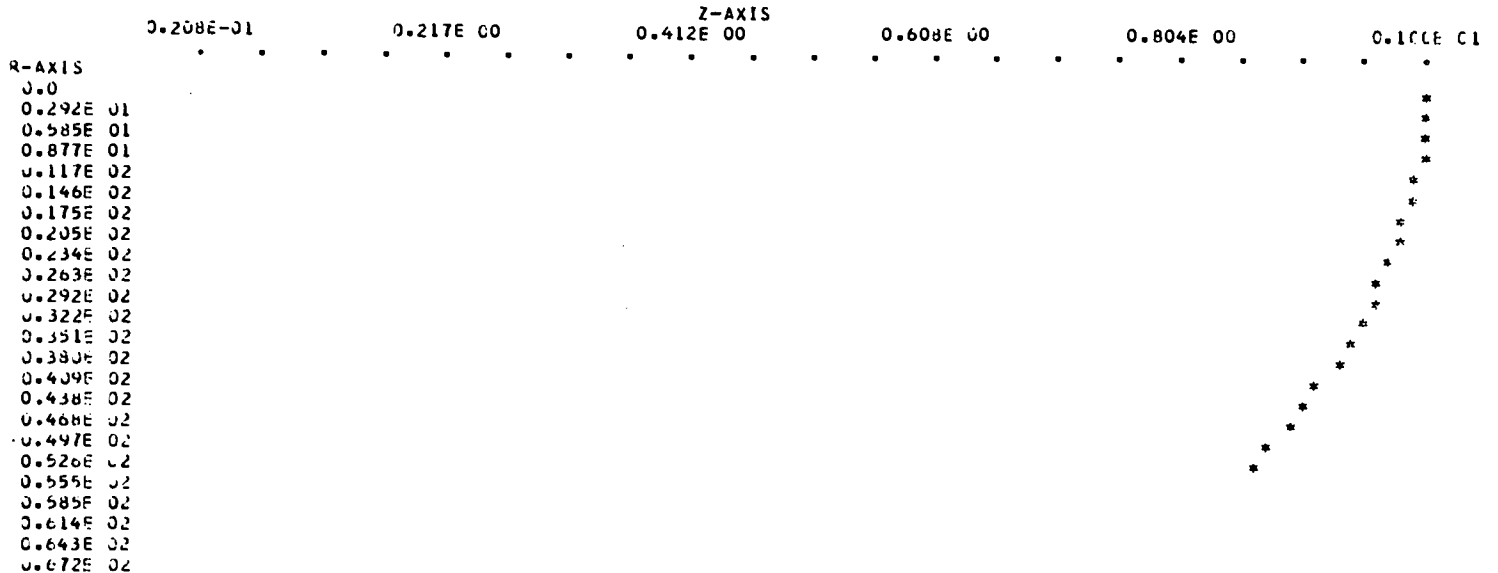
GROUP 2

0.10000E 01	0.99957E 00	0.99826E 00	0.99609E 00	0.99305E 00
0.98915E 00	0.98439E 00	0.97876E 00	0.97229E 00	0.96496E 00
0.95679E 00	0.94778E 00	0.93794E 00	0.92727E 00	0.91580E 00
0.90351E 00	0.89043E 00	0.87657E 00	0.86194E 00	0.84655E 00
0.83042E 00	0.81356E 00	0.79600E 00	0.77774E 00	0.75881E 00
0.73923E 00	0.71902E 00	0.69821E 00	0.67682E 00	0.65487E 00
0.63239E 00	0.61818E 00	0.60379E 00	0.58922E 00	0.57447E 00
0.55957E 00	0.54450E 00	0.52929E 00	0.51394E 00	0.49846E 00
0.48285E 00	0.46713E 00	0.45130E 00	0.43537E 00	0.41936E 00
0.40326E 00	0.38710E 00	0.37087E 00	0.35460E 00	0.33828E 00
0.32192E 00	0.30930E 00	0.29666E 00	0.28401E 00	0.27137E 00
0.25873E 00	0.24609E 00	0.23347E 00	0.22087E 00	0.20828E 00
0.19572E 00	0.18319E 00	0.17069E 00	0.15822E 00	0.14578E 00
0.13338E 00	0.12100E 00	0.10866E 00	0.96349E-01	0.84124E-01
0.72109E-01	0.58245E-01	0.44281E-01	0.30263E-01	0.16029E-01

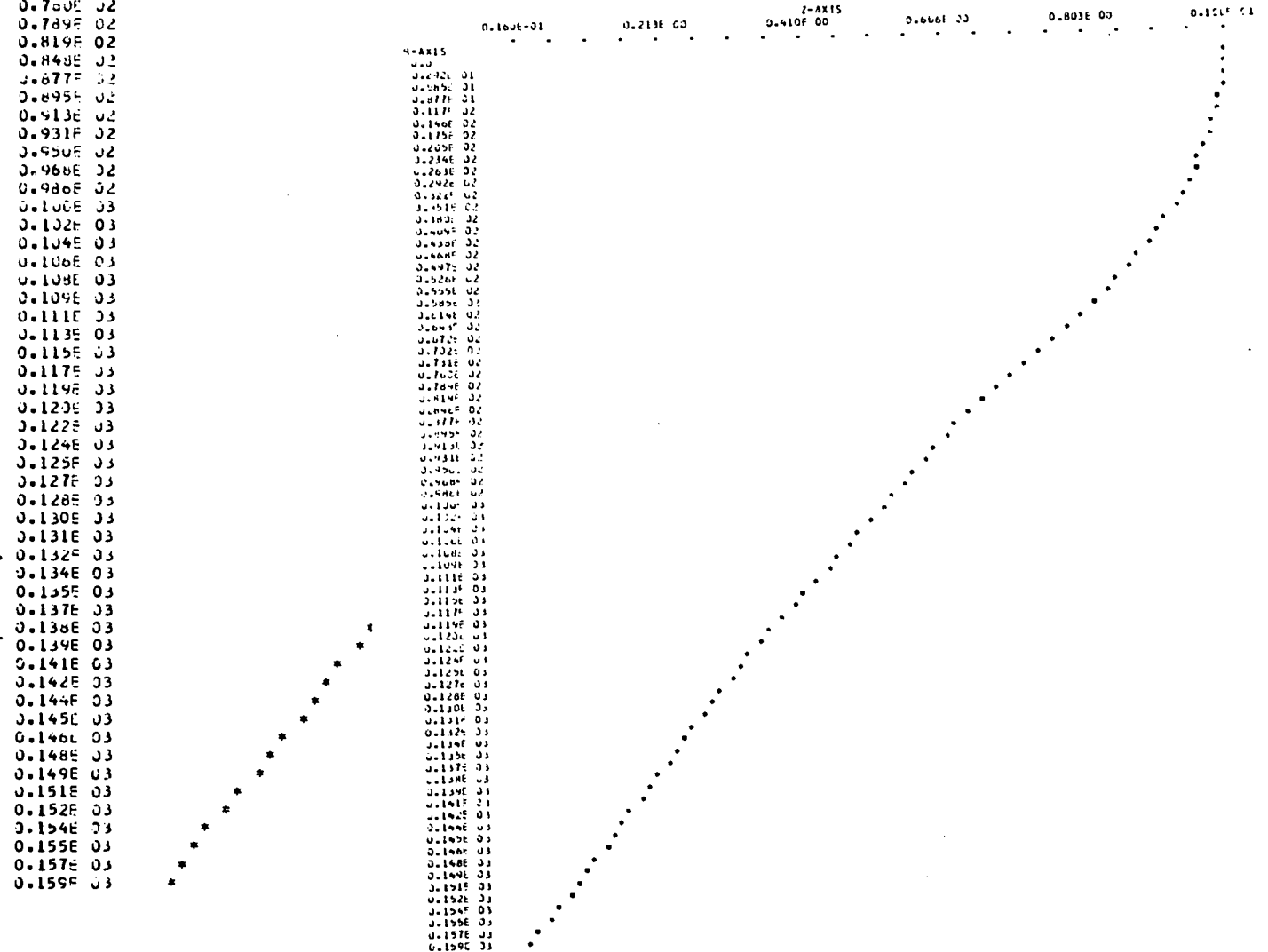
GROUP 3

0.10000E 01	0.99957E 00	0.99826E 00	0.99609E 00	0.99305E 00
0.98915E 00	0.98438E 00	0.97876E 00	0.97228E 00	0.96495E 00
0.95678E 00	0.94776E 00	0.93792E 00	0.92725E 00	0.91577E 00
0.90349E 00	0.89041E 00	0.87654E 00	0.86191E 00	0.84652E 00
0.83038E 00	0.81352E 00	0.79596E 00	0.77770E 00	0.75877E 00
0.73918E 00	0.71897E 00	0.69816E 00	0.67677E 00	0.65482E 00
0.63234E 00	0.61813E 00	0.60373E 00	0.58916E 00	0.57442E 00
0.55951E 00	0.54445E 00	0.52924E 00	0.51388E 00	0.49840E 00
0.48279E 00	0.46707E 00	0.45125E 00	0.43532E 00	0.41931E 00
0.40321E 00	0.38705E 00	0.37083E 00	0.35455E 00	0.33823E 00
0.32188E 00	0.30925E 00	0.29662E 00	0.28397E 00	0.27133E 00
0.25869E 00	0.24606E 00	0.23344E 00	0.22084E 00	0.20826E 00
0.19571E 00	0.18319E 00	0.17073E 00	0.15835E 00	0.14611E 00
0.13414E 00	0.12274E 00	0.11257E 00	0.10505E 00	0.10336E 00
0.11448E 00	0.13177E 00	0.11642E 00	0.77920E-01	0.23764E-01

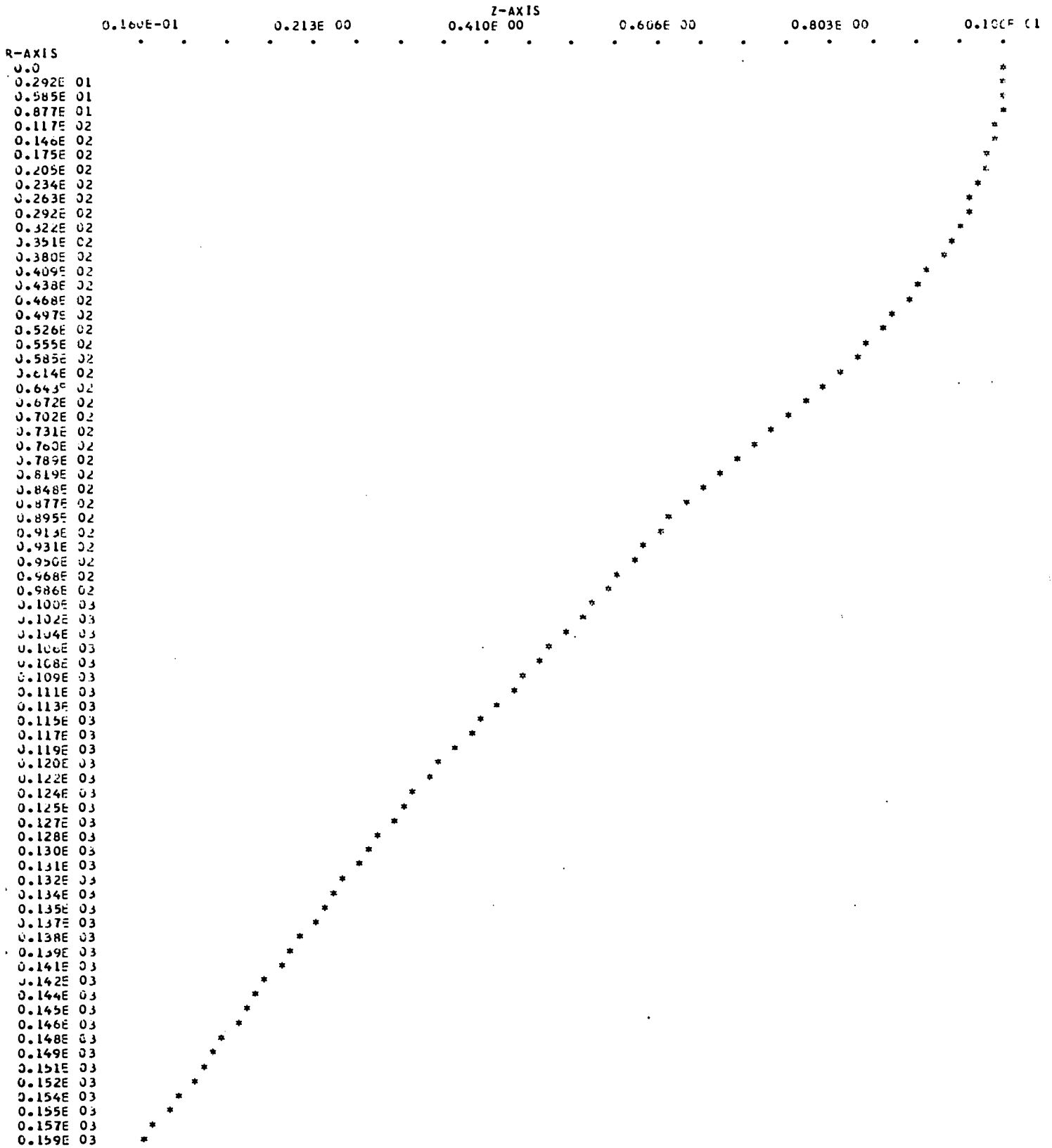
PLOT OF GROUP 1



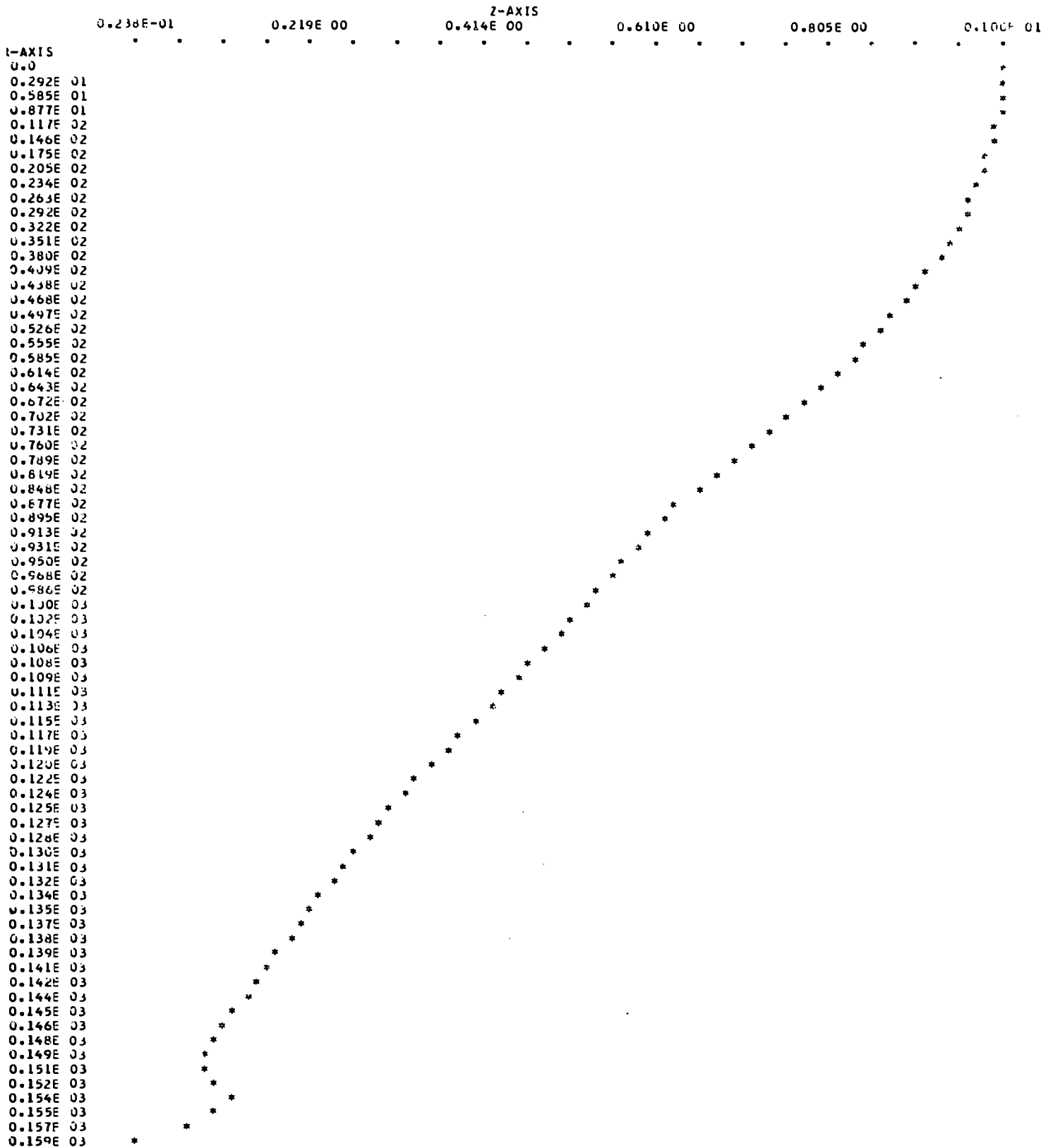
PLOT OF GROUP 2



PLOT OF GROUP 2



PLOT OF GROUP 3



RATIOS OF FLUXES TO THE SLOW FLUX

GROUP 1 / GROUP 3

0.33671E 01	0.33671E 01	0.33671E 01	0.33671E 01	0.33671E 01
0.33671E 01	0.33671E 01	0.33672E 01	0.33672E 01	0.33672E 01
0.33673E 01	0.33673E 01	0.33673E 01	0.33674E 01	0.33674E 01
0.33675E 01	0.33675E 01	0.33676E 01	0.33677E 01	0.33677E 01
0.33678E 01	0.33679E 01	0.33679E 01	0.33680E 01	0.33680E 01
0.33681E 01	0.33682E 01	0.33683E 01	0.33683E 01	0.33684E 01
0.33685E 01	0.33685E 01	0.33686E 01	0.33686E 01	0.33686E 01
0.33687E 01	0.33687E 01	0.33688E 01	0.33688E 01	0.33688E 01
0.33689E 01	0.33689E 01	0.33689E 01	0.33690E 01	0.33690E 01
0.33690E 01	0.33691E 01	0.33691E 01	0.33691E 01	0.33692E 01
0.33692E 01	0.33692E 01	0.33693E 01	0.33693E 01	0.33693E 01
0.33693E 01	0.33694E 01	0.33694E 01	0.33694E 01	0.33695E 01
0.33695E 01	0.33695E 01	0.33691E 01	0.33677E 01	0.33639E 01
0.33533E 01	0.33257E 01	0.32559E 01	0.30873E 01	0.27158E 01
0.20403E 01	0.13819E 01	0.11931E 01	0.13099E 01	0.29438E 01

GROUP 2 / GROUP 3

0.27314E 01	0.27314E 01	0.27314E 01	0.27314E 01	0.27314E 01
0.27314E 01	0.27314E 01	0.27314E 01	0.27314E 01	0.27314E 01
0.27314E 01	0.27314E 01	0.27314E 01	0.27314E 01	0.27315E 01
0.27315E 01	0.27315E 01	0.27315E 01	0.27315E 01	0.27315E 01
0.27315E 01	0.27315E 01	0.27315E 01	0.27315E 01	0.27316E 01
0.27316E 01	0.27316E 01	0.27316E 01	0.27316E 01	0.27316E 01
0.27316E 01	0.27316E 01	0.27316E 01	0.27316E 01	0.27317E 01
0.27317E 01	0.27317E 01	0.27317E 01	0.27317E 01	0.27317E 01
0.27317E 01	0.27317E 01	0.27317E 01	0.27317E 01	0.27317E 01
0.27317E 01	0.27317E 01	0.27317E 01	0.27317E 01	0.27317E 01
0.27317E 01	0.27318E 01	0.27318E 01	0.27318E 01	0.27318E 01
0.27318E 01	0.27318E 01	0.27318E 01	0.27318E 01	0.27317E 01
0.27316E 01	0.27314E 01	0.27308E 01	0.27292E 01	0.27254E 01
0.27159E 01	0.26927E 01	0.26365E 01	0.25052E 01	0.22230E 01
0.17205E 01	0.12074E 01	0.10389E 01	0.10608E 01	0.18423E 01

PLOT OF GROUP 1 / GROUP 3

	0.119E 01	0.163E 01	0.206E 01	0.250E 01	0.293E 01	0.337E 01
Z-AXIS						
0.0						*
0.292E 01						*
0.585E 01						*
0.877E 01						*
0.117E 02						*
0.146E 02						*
0.175E 02						*
0.205E 02						*
0.234E 02						*
0.263E 02						*
0.292E 02						*
0.322E 02						*
0.351E 02						*
0.380E 02						*
0.409E 02						*
0.438E 02						*
0.468E 02						*
0.497E 02						*
0.526E 02						*
0.555E 02						*
0.585E 02						*
0.614E 02						*
0.643E 02						*
0.672E 02						*
0.702E 02						*
0.731E 02						*
0.760E 02						*
0.789E 02						*
0.819E 02						*
0.848E 02						*
0.877E 02						*
0.895E 02						*
0.913E 02						*
0.931E 02						*
0.950E 02						*
0.968E 02						*
0.986E 02						*
0.100E 03						*
0.102E 03						*
0.104E 03						*
0.106E 03						*
0.108E 03						*
0.109E 03						*
0.111E 03						*
0.113E 03						*
0.115E 03						*
0.117E 03						*
0.119E 03						*
0.120E 03						*
0.122E 03						*
0.124E 03						*
0.125E 03						*
0.127E 03						*
0.128E 03						*
0.130E 03						*
0.131E 03						*
0.132E 03						*
0.134E 03						*
0.135E 03						*
0.137E 03						*
0.138E 03						*
0.139E 03						*
0.141E 03						*
0.142E 03						*
0.144E 03						*
0.145E 03						*
0.146E 03						*
0.148E 03						*
0.149E 03						*
0.151E 03						*
0.152E 03			*			*
0.154E 03	*	*				*
0.155E 03						*
0.157E 03						*
0.159E 03					*	*

PLOT OF GROUP 2 / GROUP 3

0.104E 01 0.138E 01 Z-AXIS 0.172E 01 0.205E 01 0.239E 01 0.272E 01

```

-AXIS
J.C
0.292E 01
0.585E 01
0.877E 01
0.117E 02
0.146E 02
0.175E 02
0.205E 02
0.234E 02
0.263E 02
0.292E 02
0.322E 02
0.351E 02
0.380E 02
0.409E 02
0.438E 02
0.468E 02
0.497E 02
0.526E 02
0.555E 02
0.585E 02
0.614E 02
0.643E 02
0.672E 02
0.702E 02
0.731E 02
0.760E 02
0.789E 02
0.819E 02
0.848E 02
0.877E 02
0.895E 02
0.913E 02
0.931E 02
0.950E 02
0.968E 02
0.986E 02
0.100E 03
0.102E 03
0.104E 03
0.106E 03
0.108E 03
0.109E 03
0.111E 03
0.113E 03
0.115E 03
0.117E 03
0.119E 03
0.120E 03
0.122E 03
0.124E 03
0.125E 03
0.127E 03
0.128E 03
0.130E 03
0.131E 03
0.132E 03
0.134E 03
0.135E 03
0.137E 03
0.138E 03
0.139E 03
0.141E 03
0.142E 03
0.144E 03
0.145E 03
0.146E 03
0.148E 03
0.149E 03
0.151E 03
0.152E 03
0.154E 03
0.155E 03
0.157E 03
0.159E 03

```

GROUP 1 / GROUP 2

0.12327E 01	0.12327E 01	C.12327E 01	0.12327E 01	C.12327E 01
0.12327E 01	0.12327E 01	C.12328E 01	0.12328E 01	0.12328E 01
0.12328E 01	0.12328E 01	0.12328E 01	0.12328E 01	0.12328E 01
0.12329E 01	0.12329E 01	0.12329E 01	0.12329E 01	C.12329E 01
0.12329E 01	0.12330E 01	C.12330E 01	0.12330E 01	0.12330E 01
0.12330E 01	0.12331E 01	0.12331E 01	0.12331E 01	0.12331E 01
0.12331E 01	0.12331E 01	0.12332E 01	0.12332E 01	0.12332E 01
0.12332E 01	0.12332E 01	C.12332E 01	0.12332E 01	0.12332E 01
0.12333E 01	0.12333E 01	0.12333E 01	0.12333E 01	0.12333E 01
C.12333F 01	0.12333E 01	C.12333E 01	0.12333E 01	0.12333E 01
0.12333E 01	0.12334E 01	0.12334E 01	C.12334E 01	0.12334E 01
0.12334E 01	0.12334E 01	C.12334E 01	0.12334E 01	C.12335E 01
0.12335E 01	0.12335E 01	0.12337E 01	0.12339E 01	0.12343E 01
0.12347E 01	0.12351E 01	0.12349E 01	0.12324E 01	0.12217E 01
0.11858E 01	0.11446E 01	0.11485E 01	C.12348E 01	0.15979E 01

- 55 -
 PLGT CF GROUP 1 / GROUP 2

	0.114E 01	0.124E 01	Z-AXIS 0.133E 01	0.142E 01	0.151E 01	0.160E 01
R-AXIS						
0.0		*				
0.292E 01		*				
0.585E 01		*				
0.877E 01		*				
0.117E 02		*				
0.146E 02		*				
0.175E 02		*				
0.205E 02		*				
0.234E 02		*				
0.263E 02		*				
0.292E 02		*				
0.322E 02		*				
0.351E 02		*				
0.380E 02		*				
0.409E 02		*				
0.438E 02		*				
0.467E 02		*				
0.497E 02		*				
0.526E 02		*				
0.555E 02		*				
0.585E 02		*				
0.614E 02		*				
0.643E 02		*				
0.672E 02		*				
0.702E 02		*				
0.731E 02		*				
0.760E 02		*				
0.789E 02		*				
0.818E 02		*				
0.848E 02		*				
0.877E 02		*				
0.906E 02		*				
0.935E 02		*				
0.965E 02		*				
0.986E 02		*				
0.100E 03		*				
0.102E 03		*				
0.104E 03		*				
0.106E 03		*				
0.108E 03		*				
0.109E 03		*				
0.111E 03		*				
0.113E 03		*				
0.115E 03		*				
0.117E 03		*				
0.119E 03		*				
0.120E 03		*				
0.122E 03		*				
0.124E 03		*				
0.125E 03		*				
0.127E 03		*				
0.128E 03		*				
0.130E 03		*				
0.131E 03		*				
0.132E 03		*				
0.134E 03		*				
0.135E 03		*				
0.137E 03		*				
0.138E 03		*				
0.139E 03		*				
0.141E 03		*				
0.142E 03		*				
0.144E 03		*				
0.145E 03		*				
0.146E 03		*				
0.148E 03		*				
0.149E 03		*				
0.151E 03		*				
0.152E 03	*	*				
0.154E 03	*	*				
0.155E 03	*	*				
0.157E 03		*				
0.159E 03		*				

POWER OUTPUT IN REGIONS 1,2,3

0.13406E 10 0.77863E 09 0.32180E 09

POWER DENSITY IN REGIONS 1,2,3 (KW/LITER)

0.15171E 03 0.13348E 01 0.54903E 00